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Republic of Latvia

Cabinet

Regulation No. 129

Adopted 19 March 2002

Requirements for Operations with Radioactive Waste and Materials Related Thereto

Issued pursuant to Section 27, Paragraph three of the Law
On Radiation Safety and Nuclear Safety

I. General Provisions

1. These Regulations determine requirements for operations with radioactive waste and materials related thereto.
2. Materials related to radioactive waste are materials, installations and objects which have come into contact with radioactive waste and the total or specific radioactivity of which is less than the limits prescribed by Cabinet Regulations No 288 of 3 July 2001, Regulations on Operations with Sources of Ionising Radiation for which Special Permit (Licence) or Permit is not Required (hereinafter — limits prescribed by regulatory enactments for the non-exceeding of which a special permit (licence) or licence is not required).
3. An operator whose operations with sources of ionising radiation may produce radioactive waste (hereinafter — operator) when planning the operations referred to shall also plan operations with the relevant radioactive waste.
4. In conformity with the amount of radioactivity of radioactive waste, radioactive waste shall be classified as follows:
 - 4.1. radioactive waste as a result of the radioactive decay of which the total or specific radioactivity is smaller than the limits prescribed by regulatory enactments for the non-exceeding of which a special permit (licence) or permit is not required (hereinafter — unusable materials); and
 - 4.2. radioactive waste the total or specific radioactivity of which is greater than the limits prescribed by regulatory enactment for the non-exceeding of which a special permit (licence) or permit is not required.
5. In conformity with the half-life period radioactive waste shall be classified as follows:

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5.1. radioactive waste the half-life of which does not exceed 30 days and in which after radionuclide decay no long-lived daughter products are generated (hereinafter — short-lived radioactive waste); and

5.2. radioactive waste the half-life of which exceeds 30 days or in which after radionuclide decay long-lived daughter products are generated (hereinafter — long-lived radioactive waste).

6. In conformity with the heat generation rate radioactive waste shall be classified as follows:

6.1. radioactive waste the heat generation rate as a result of radioactive decay of which is less than 2 kW per one cubic metre of radioactive waste (hereinafter — low and intermediate radioactivity waste); and

6.2. radioactive waste the heat generation rate as a result of radioactive decay of which is greater or equal to 2 kW per one cubic metre of radioactive waste (hereinafter — high radioactivity waste).

7. Materials related to radioactive waste shall be classified in the following groups:

7.1. material related to radioactive waste which is removed from State supervision and control in the field of radiation safety and nuclear safety (hereinafter — State supervision) because its specific radioactivity is less than the minimum significant specific radioactivity prescribed in Annex 1 of these Regulations and the ionising radiation dose received as a result of the utilisation of such material:

7.1.1. is less than 100 μSv per year — for an individual user (for the whole body);

7.1.2. is less than 5 mSv per year — for an individual user (for the skin surface);

and

7.1.3. is less than 1 mSv per year — for a collective dose (for the critical group of population); and

7.2. material related to radioactive waste the specific radioactivity of which is greater than the minimum significant specific radioactivity prescribed in Annex 1 of these Regulations, but smaller than the limits prescribed by regulatory enactments for the non-exceeding of which a special permit (licence) or permit is not required.

II. Requirements for Recording of Radioactive Waste and Materials Related Thereto

8. The work supervisor in the operator-controlled area (hereinafter — work supervisor of the operator) shall perform recording of radioactive waste and materials related to radioactive waste. The recording referred to shall ensure:

8.1. a possibility to select the most suitable management methods for the relevant radioactive waste and materials related thereto; and

8.2. a possibility to make a documentary check of the time and manner by which:

8.2.1. radioactive waste arrives at an undertaking (company) which in conformity with the articles of association deals with the disposal and management of radioactive waste (management, treatment, preparation for long-term storage or disposal, storage, long-term storage and disposal of radioactive waste) (hereinafter — a radioactive waste disposal or management undertaking);

8.2.2. spent sources of ionising radiation are sent back to the manufacturers thereof;

8.2.3. unusable materials are dispersed in the environment or disposed of together with municipal or hazardous waste; and

8.2.4. materials related to radioactive waste are repeatedly utilised.

9. Records of radioactive waste shall include information regarding:

9.1. radioactive waste which has been produced while carrying out operations with sources of ionising radiation (quantity (in units by mass and volume), state of aggregation, total and specific radioactivity and radionuclides);

9.2. radioactive waste located in the special premises or premises for the collection, treatment, packing and storage of radioactive waste (hereinafter — temporary radioactive waste repository);

9.3. radioactive waste which has been transferred for disposal and long-term storage, including information included in the labelling of the packaging suited for transportation and disposal of radioactive waste (hereinafter — radioactive waste package);

9.4. spent sources of ionising radiation sent back to the manufacturer of ionising radiation sources;

9.5. unusable material dispersed in the environment:

9.5.1. main radionuclides;

9.5.2. specific and total radioactivity (Bq/g and Bq);

9.5.3. date of dispersal;

9.5.4. results of dispersal control to prove that the permissible amounts of radionuclides to be dispersed in the environment prescribed by regulatory enactments have not been exceeded; and

9.5.5. each dispersed portion and a summary thereof for a year; and

9.6. individual numbers assigned to the radioactive waste packages.

10. Records of the materials related to radioactive waste shall include the following information:

10.1. name;

10.2. main radionuclides;

10.3. radioactive contamination the specific radioactivity of which radioactive substance on a surface exceeds 0.4 Bq/cm^2 for beta radiation sources, gamma radiation sources and low toxicity alpha radiation sources or 0.04 Bq/cm^2 for other alpha radiation sources if such radioactive substances:

10.3.1. cannot be removed from the surface with the usual decontamination methods (hereinafter — fixed radioactive contamination); or

10.3.2. may be removed from the surface with the usual decontamination methods (hereinafter — non-fixed radioactive contamination);

10.4. specific and total radioactivity (Bq/g and Bq);

10.5. types of use; and

10.6. institution or undertaking (company) to which the material related to radioactive waste has been transferred for further use and the date of transfer.

11. The work supervisor of the operator shall:

11.1. each year by 31 January send to the Radiation Safety Centre (hereinafter — Centre) the information referred to in Paragraphs 9 and 10 of these Regulations for the previous calendar year; and

11.2. each year by 31 December send to the Centre and the radioactive waste disposal and management undertaking an informative letter regarding those operations with sources of ionising radiation, which may cause generation of radioactive waste in the next calendar year.

12. The informative letter referred to in Sub-paragraph 11.2 of these Regulations shall specify:

12.1. the information regarding the operator, the work supervisor of the operator and the person responsible for radioactive waste;

12.2. the amount of the planned radioactive waste, its radioactivity and form — radionuclides in the form of open radiation sources or radionuclides in the form of sealed radiation sources;

12.3. a list of the measuring instruments utilised in the measurement of the ionising radiation dose rate, measurements of non-fixed and fixed radioactive contamination and determination of the qualitative and quantitative content of radionuclides;

12.4. a description of the temporary radioactive waste repository; and

12.5. a description of the planned operations with radioactive waste, including disposal or long-term storage and specifying the measures planned in order to meet the requirements of the radioactive waste management or disposal undertaking.

III. Requirements for Operations with Radioactive Waste and Materials Related Thereto in Places of Generation Thereof

13. The operator shall ensure:

13.1. financial, technical and worker resources for safe operations with radioactive waste and materials related thereto;

13.2. minimum total or specific radioactivity and volume of radioactive waste; and

13.3. transfer of radioactive waste for disposal.

14. Sorting, treatment and storage of radioactive waste shall be performed by dividing the waste into the following groups:

14.1. gaseous radioactive waste (without applying storage);

14.2. liquid radioactive waste;

14.3. solid radioactive waste;

14.4. spent sealed radiation sources; and

14.5. biological radioactive waste.

15. The work supervisor of the operator shall ensure that:

15.1. each radioactive waste container contains radioactive waste of one group only;

15.2. in a radioactive waste container the following are not mixed together:

15.2.1. treated and untreated radioactive waste;

15.2.2. short-lived radioactive waste and long-lived radioactive waste;

15.2.3. low and intermediate radioactivity waste and high radioactivity waste; and

15.2.4. radioactive waste referred to in Sub-paragraph 4.1 and 4.2 of these

Regulations;

15.3. radioactive waste is treated, transforming it into such chemical and physical form, which is safe and appropriate for storage and transportation of radioactive waste;

15.4. liquid radioactive waste is neutralised;

15.5. radioactive waste does not contain explosive or inflammable materials or compressed gases;

15.6. radioactive waste which contains rotting or decomposing substances or objects (including food or dead bodies of animals) is treated protecting it from biological decomposition processes or is stored frozen;

15.7. radioactive waste containing chemically or biologically hazardous materials and materials which may cause dangerous infectious diseases or in any other manner endanger epidemiological safety is treated maximally reducing those hazardous properties of radioactive waste which are not related to its radioactivity; and

15.8. prior to disposal of radioactive waste the necessity to perform additional treatment is as small as possible.

16. The operator shall ensure that the radioactive waste container that is utilised in a temporary radioactive waste repository is:

16.1. durable;

16.2. compatible with the radioactive waste placed therein; and

16.3. safely filled and emptied.

17. If a radioactive waste container is filled, the work supervisor of the operator shall ensure that the radioactive waste container:

17.1. is labelled with the following information:

17.1.1. the radiation warning sign;

17.1.2. the date on which the radioactive waste container was filled;

17.1.3. the type of radioactive waste generation;

17.1.4. a symbol or name of radionuclides and the total and specific radioactivity thereof (Bq and Bq/g), the date of its measurement;

17.1.5. the ionising radiation dose rate at the distance of 1 cm and 1 m from the surface;

17.1.6. the group of radioactive waste and an indication whether it is short-lived or long-lived radioactive waste and whether it is low and intermediate or high radioactivity waste; and

17.1.7. the name, legal address and telephone number of the radioactive waste packing undertaking (company); and

17.2. is without delay conveyed to the temporary radioactive waste repository.

18. If it is necessary to store and later transfer for disposal radioactive waste the size of which is greater than the radioactive waste storage containers, the work supervisor of the operator shall ensure crushing of the radioactive waste to the appropriate size.

19. Solid radioactive waste shall be accumulated in plastic bags and together with this outer packaging shall be placed:

19.1. into a 0.1 m³ steel drum if the amount of radioactive waste per year does not exceed 1 m³ and the total or specific radioactivity thereof does not exceed the values specified in Annex 2 of these Regulations;

19.2. into special reinforced concrete containers which meet the requirements prescribed by Cabinet Regulation No 307 of 3 July 2001, Regulations on Protection Against Ionising Radiation while Transporting Radioactive Materials in respect of IP-3 or type A packages if:

19.2.1. the amount of radioactive waste per year exceeds 1 m³; or

19.2.2. the size of radioactive waste is large and it cannot be emplaced into a 0.1 m³ steel drum.

20. Radioactive waste shall be prepared for transportation, labelled and transported in compliance with the requirements prescribed by Cabinet Regulation No 307 of 3 July 2001, Regulations on Protection Against Ionising Radiation while Transporting Radioactive Materials.

21. The work supervisor of the operator shall ensure:

21.1. preparation of radioactive waste for transportation and labelling thereof;

21.2. examination of a radioactive waste package so that it meets the following requirements:

21.2.1. non-fixed radioactive contamination on the external surface of the package for beta radiation sources, gamma radiation sources and low toxicity alpha radiation sources does not exceed 4 Bq/cm² per 300 cm² of the available surface, but for other alpha radiation sources — 0.4 Bq/cm² per 300 cm² of the available surface; and

21.2.2. the ionising radiation dose rate does not exceed the value permissible for the relevant package and 2 mSv/h at any point of the external surface.

22. If after elimination of radiation emergency consequences the amount of radioactive waste exceeds three cubic metres, it is permitted to accumulate the waste in 0.1 m³ steel drums which shall be transported as exceptional packages in conformity with the requirements prescribed by Cabinet Regulation No 307 of 3 July 2001, Regulations on Protection Against Ionising Radiation while Transporting Radioactive Materials. In such case the work supervisor of the operator shall specify in the labelling of the package at least the following:

22.1. the main radionuclides;

22.2. the assessed maximum total radioactivity of the radioactive content expressed in the measurement units of the SI system or at least the ionising radiation dose rate on the surface of the drum and at a distance of 1 m, as well as the time of measurement of such values; and

22.3. the category of the package — I-white, II-yellow or III-yellow.

23. Radioactive waste shall be transported by the specialised transport of the radioactive waste disposal or management undertaking or by a means of transport of the operator (if the operator has an appropriate special permit (licence) or a permit for transportation of radioactive waste) which meets the requirements prescribed by Cabinet Regulation No 401 of 7 December 1999, Regulations on Road Haulage of Dangerous Goods.

24. Large quantities of radioactive waste which has been generated when eliminating the consequences of radiation emergencies, if the total and specific radioactivity thereof does not exceed the values specified in Annex 2 of these Regulations, may be conveyed as solid low specific radioactivity waste (except for powder) in LSA-III 0.1 m³ steel drums which prior to disposal of radioactive waste together with the radioactive waste shall be placed in 0.2 m³ steel

drums or reinforced concrete containers and the empty space (also beneath and above the 0.1 m³ steel drum) shall be filled with concrete.

25. If the operator has been declared insolvent and cannot manage radioactive waste or prepare it for transfer for disposal or long-term storage, the Centre shall inform the relevant local government of the potential danger to the inhabitants of this local government and together with the operator shall ensure the possibility of transferring for disposal all radioactive waste and sources of ionising radiation which are not required for further operations or whose safety no longer meets the requirements prescribed by regulatory enactments.

26. The Centre shall issue a permit to repeatedly utilise materials related to radioactive waste which are not subject to State supervision outside the operator-controlled area.

27. Materials related to radioactive waste, which may be repeatedly utilised without additional treatment, shall be removed from State supervision taking into account the radioactive contamination thereof.

28. In calculating the maximum permissible fixed and non-fixed radioactive contamination on a surface the values of ionising radiation doses prescribed in Annex 1 of these Regulations or Sub-paragraph 7.1 of these Regulations shall be used as criteria. These criteria shall refer to the total fixed and non-fixed radioactive contamination.

29. When using radioactive contamination on a surface as a criterion, the average level of radioactive contamination shall be determined for 300 cm² of the available surface of the material or for the whole surface of the material if it is smaller than 300 cm². The work supervisor of the operator shall ensure that the non-fixed radioactive contamination is as small as possible.

30. When using specific radioactivity as a criterion, the average specific radioactivity for each 100 kg of the total mass of radioactive materials or for the whole amount of materials if the total mass of radioactive materials is less than 100 kg shall be determined.

IV. Requirements for Storage of Radioactive Waste and Materials Related Thereto

31. In conformity with the purpose of radioactive waste storage the following types of storage shall be distinguished:

31.1. storage of radioactive waste until the time when due to decay its specific radioactivity has decreased to the minimum significant specific radioactivity and it cannot be subjected to the State supervision (hereinafter — storage until decay);

31.2. storage of radioactive waste until its treatment is carried out;

31.3. storage of treated radioactive waste prior to shipment to a radioactive waste disposal or management undertaking;

31.4. storage of a spent sealed radiation source prior to sending it back to the manufacturer; and

31.5. long-term storage of radioactive waste until:

31.5.1. such radioactive waste repository has been created which is situated in geologically stable formations in the depth of at least 100 m from the earth's surface (hereinafter — geological repository); or

31.5.2. a possibility has been found to send radioactive waste for processing in a solid state in conformity with the packaging chosen in order to be able to safely transport and dispose it (hereinafter — processing) outside Latvia.

32. In a temporary radioactive waste repository and in an engineering construction located at little depth from the earth's surface or created on the earth's surface (hereinafter — near-surface repository) multiple protection barriers shall be utilised to ensure safe storage of radioactive waste.

33. For the long-term storage of radioactive waste only a near-surface repository of a radioactive waste management or disposal undertaking may be utilised. As safe storage criteria the following shall be utilised:

33.1. the impact of the relevant repository on environmental pollution, taking into account the natural radioactivity level — the specific radioactivity of radionuclides in materials and starting materials encountered in the relevant territory on average (with the condition that its amount has not significantly increased after some radiation emergency); and

33.2. the effect of long-term stored radioactive waste on a worker who carries out operations with sources of ionising radiation in the relevant repository.

34. In storing radioactive waste the following requirements shall be observed:

34.1. for each specific group of radioactive waste storage shall be ensured separately from other groups, taking into account also the requirements specified in Sub-paragraph 15.2;

34.2. for non-treated radioactive waste temporary storage shall be planned for not longer than a year and during this time radioactive waste shall be treated, packaged and transferred for disposal or long-term storage;

34.3. the radioactive waste container, if possible, shall not be moved in the repository until the transfer for disposal or long-term storage;

34.4. radioactive waste shall be stored separately from those sources of radioactive radiation which can be utilised for further operations; and

34.5. radioactive waste shall be stored in containers, which are suited for the relevant type of storage and are appropriately labelled.

35. If it is intended to store a group of radioactive waste until decay, in order to reduce the amount of radioactive waste to be sent for disposal, a special compartment in the temporary radioactive waste repository shall be created which is intended for storage until decay and in which emplacement of other groups of radioactive waste or materials related to radioactive waste is prohibited.

36. Storage until decay shall be applied to short-lived radioactive waste, as well as other radioactive waste if the operator ensures that it shall be stored as long as necessary so that after radionuclide decay its radioactivity does not exceed the minimum significant radioactivity.

37. Materials related to radioactive waste which contain short-lived radioactive waste shall be stored in a temporary radioactive waste repository until its specific radioactivity as a result of radionuclide decay becomes smaller than the minimum significant radioactivity.

38. The operator shall ensure that storage until decay does not exceed one year but in separate cases, if it is provided for in the special permit (licence) or permit for operations with sources of ionising radiation, the storage time until radionuclide decay may be five years.

39. The compartment which is intended for storage until decay shall provide a possibility to disperse under controlled circumstances the unusable materials generated upon radionuclide decay, taking into account the permissible amounts of radionuclides for dispersal specified in regulatory enactments.

40. A temporary radioactive waste repository:

40.1. shall be so created that all radioactive waste which may be generated within a year can be stored separately by groups;

40.2. shall be provided with protection against unauthorised entry therein and the possibility to ascertain that no unauthorised operations with radioactive waste are carried out;

40.3. shall be equipped with installations and materials for protection against ionising radiation;

40.4. shall be so created that all surfaces are made of materials, which are easy to decontaminate or are covered with such materials;

40.5. shall be provided with air purification systems and exhaust gas monitoring facilities if during storage radioactive gases may be released from radioactive waste or the air in the repository may be polluted with radioactive aerosols; and

40.6. shall be so created that all operations are carried out under safe conditions and it is possible to easily transfer radioactive waste containers so as to transport them to a radioactive waste disposal or management undertaking.

41. Flammable or combustible radioactive waste shall be stored separately from other radioactive waste, ensuring the fulfilment of the necessary fire-safety requirements and ensuring the processing thereof in a maximally short period of time.

42. Liquid radioactive waste shall be stored in hermetically sealed double-containers. The work supervisor of the operator shall ensure that the volume of the outer radioactive waste container is sufficient for safe storage of all liquid radioactive waste if the internal container loses leak-tightness.

V. Requirements for Dispersal of Unusable Materials and Materials Related to Radioactive Waste in the Environment

43. Prior to dispersal of unusable materials or materials related to radioactive materials in the environment the work supervisor of the operator shall:

43.1. check whether the chemical and biological properties of unusable materials to be dispersed in the environment allow the application of the dispersal method;

43.2. check whether there is the necessary guarantee to prevent the exceeding of the permissible amounts of radionuclides for dispersal (for each portion of dispersal or day and year) prescribed in regulatory enactments regarding protection against ionising radiation;

43.3. check whether at the point of dispersal a possibility to dilute the radioactive waste is ensured, so that its total and specific radioactivity does not exceed the permissible amounts of radionuclides for dispersal in the environment prescribed in regulatory enactments;

43.4. make calculations to ascertain that the ionising radiation dose limits specified in Sub-paragraph 7.1 of these Regulations are not exceeded; and

43.5. inform the Centre:

43.5.1. every year by 15 December — regarding the planned dispersal of radioactive waste in the environment during the next calendar year; and

43.5.2. at regular intervals but not less than four times a year (by 15 April, 15 July, 15 October and 15 January) — regarding the dispersal operations carried out in the previous quarter.

44. When releasing liquid unusable materials into the sewerage system together with household waste water, the work supervisor of the operator shall ensure that:

44.1. at the waste water discharge site in the sewerage system they are diluted, reducing the concentration at least 10 times;

44.2. water solutions do not contain insoluble admixtures;

44.3. pH of the unusable materials is neutral; and

44.4. unusable materials are released only at one specially equipped site in which before and after their discharge water flows from the water main for at least one minute with a normal average flow.

45. Solid unusable materials or materials related to radioactive waste may be disposed of in a municipal or hazardous waste landfill site or dump if:

45.1. they do not contain explosive, inflammable materials and compressed gases;

45.2. there is no non-fixed radioactive contamination on the surface of the unusable material packaging but the fixed radioactive contamination does not exceed 0.4 Bq/cm^2 for beta radiation sources, gamma radiation sources and low toxicity alpha radiation sources or 0.04 Bq/cm^2 for other alpha radiation sources; and

45.3. they do not contain a sealed radiation source.

46. When dispersing gaseous unusable materials in the atmosphere the work supervisor of the operator shall ensure that:

46.1. the ventilation power disperses them reducing the concentration at least 10 times; and

46.2. all unusable materials are released into the atmosphere at one specially equipped site only.

47. If regulatory enactments do not specify radionuclide dispersal amounts for any of the radionuclides, the work supervisor of the operator shall prepare a temporary calculation of dispersal amounts, utilising the following conditions:

47.1. the maximum ionising radiation dose for the critical group of the population (the population who are exposed to the effect of a source of ionising radiation at a similar level and to

whom the exposure from such source is the greatest) may not exceed 100 μSv per year but the maximum average ionising radiation dose — 10 μSv per year. These values shall be utilised in calculating the dispersal limits for one solid unusable material portion or one day;

47.2. the specific radioactivity is equal to the radioactivity for which the special permit (licence) or permit for operations with sources of ionising radiation is required or smaller than that;

47.3. radionuclides get into the organism of an adult by intake thereof with water (liquid radioactive waste), food (solid radioactive waste) or by inhalation (gaseous radioactive waste); and

47.4. in numerical calculations of ionising radiation doses coefficients specified in regulatory enactments regarding protection against ionising radiation shall be used.

48. The operator shall co-ordinate the temporary dispersal amounts for radionuclides with the Centre. The Centre may permit to carry out separate operations, applying the calculated temporary dispersal amounts if the time of performance of the operations does not exceed three months.

VI. Requirements for Processing of Radioactive Waste

49. The operator in the operator's controlled area may create a radioactive waste processing site and carry out the processing thereof or, if such is provided for in the contract, transfer for processing to the radioactive waste disposal or management undertaking unprocessed radioactive waste.

50. When processing radioactive waste the requirements put forward by all the following operations shall be observed in each operation and the necessity to carry out additional treatment or processing of the previously processed radioactive waste at the radioactive waste disposal or management undertaking shall be prevented.

51. The operator of the radioactive waste disposal or management undertaking shall receive from the operator information regarding physical and chemical properties of the radioactive waste, the amount in units by mass or volume and the total or specific radioactivity.

52. The processor of radioactive waste shall be responsible for the use of appropriate processing technology and shall ensure that the external surface of the radioactive waste container is not contaminated with radioactive substances and the ionising radiation dose rate on the external surface of the package does not exceed the permissible limit.

53. When processing radioactive waste the work supervisor of the radioactive waste processor shall ensure that the level of specific radioactivity in each radioactive waste package (except for the package of a spent sealed source of radiation) is as even as possible, avoiding high specific radioactivity in any separate part of the package.

54. When processing radioactive waste such processing methods shall be selected that:

54.1. the obtained radioactive waste package is structurally stable and all radioactive materials in the package are in a chemically and mechanically stable monolithic form which restricts the spread of radioactive substances outside the radioactive waste package;

54.2. the radioactive waste package has as far as possible no empty spaces;

54.3. the radioactive waste package is not damaged within the range of the ambient temperature from -40°C to $+55^{\circ}\text{C}$; and

54.4. the radioactive waste package endures a pressure of 0.35 MPa and the vertical deformations after such check do not exceed 3%.

55. Solid radioactive waste in conformity with its total radioactivity, mass and dimensions shall be processed in the following manner:

55.1. it shall be packaged in a 0.1 m^3 steel drum which shall be emplaced in the middle of a 0.2 m^3 steel drum and the empty space among the radioactive waste shall be filled with concrete, ensuring that the whole empty space between the 0.1 m^3 and 0.2 m^3 steel drum (also beneath and above the 0.1 m^3 drum) is filled with concrete;

55.2. it shall be emplaced in a 0.2 m^3 steel drum and the empty space between the radioactive waste shall be filled with concrete, ensuring that the whole empty space between the drum and radioactive waste (also beneath and above the radioactive waste) is filled with concrete;

55.3. it shall be packaged in a 0.1 m^3 steel drum which shall be emplaced into special sockets in a reinforced concrete container; or

55.4. it shall be emplaced in a reinforced concrete container and the empty space among the radioactive waste shall be filled with concrete, ensuring that the whole empty space between the container and radioactive waste (also beneath and above the radioactive waste) is filled with concrete.

56. If the total radioactivity of radioactive waste is so great that concrete does not ensure the possibility of observing the permissible limits for ionising radiation dose rate on the package surface, additional protection layers shall be created which attenuate ionising radiation more effectively than concrete.

57. Liquid radioactive waste subsequent to its cementation shall be filled in a 0.2 m^3 steel drum. Cementation of liquid radioactive waste directly in the steel drum may be utilised as an alternative method if there is appropriate technical capacity for the performance of such work.

58. If ion exchange resins have been used for purification of liquid radioactive waste, such resins shall be processed utilising the cementation method and shall be filled into a 0.2 m^3 steel drum. Cementation of waste directly in the steel drum may be utilised as an alternative method if there is appropriate technical capacity for the performance of such work.

59. Organic liquid radioactive waste shall be processed into inorganic liquid radioactive waste, taking into account its radioactive, chemical, physical and toxic properties.

60. Organic liquid radioactive waste in small amounts may be added to liquid inorganic radioactive waste if it is experimentally proven that the cement matrix that has formed is

sufficiently stable. In such case the cemented radioactive waste shall be filled into a 0.1 m³ steel drum which shall be emplaced in a special socket in a reinforced concrete container.

61. Spent sealed gamma radiation sources in conformity with their total radioactivity, number and dimensions:

61.1. shall be emplaced into a lead container, in exceptional cases — in an industrial container where these sources of ionising radiation were located during their use, ensuring in addition that all openings and ducts of this container are made airtight with melted lead. Thereafter the container shall be emplaced in the middle of a 0.2 m³ steel drum and the empty space between the container and the drum (also beneath and above the container) shall be filled with concrete;

61.2. shall be packaged in a stainless steel container which shall be filled with lead pellets and emplaced into a special socket in a reinforced concrete container; or

61.3. shall be packaged in a stainless steel container which shall be filled with lead pellets, placed into the middle of a 0.2 m³ steel drum and the empty space (also beneath and above the container) shall be filled with concrete.

62. Spent sealed sources of beta radiation in conformity with their total radioactivity, number and dimensions:

62.1. shall be emplaced into a stainless steel or plastic container, in exceptional cases — in special plastic bags, ensuring that the spent sealed sources of radiation are packaged in at least three bags. Thereafter the radioactive container or plastic bags shall be emplaced into a 0.2 m³ steel drum and the empty space shall be filled with concrete, ensuring that the whole empty space between the plastic bag or container and the drum (also beneath and above the bag or container) is filled with concrete; or

62.2. shall be packaged in a stainless steel container, which shall be filled with heated quartz sand and shall be placed in a special socket in a reinforced concrete container.

63. Spent sealed radium containing radiation sources shall be emplaced into a copper ampoule, which shall be hermetically sealed. The copper ampoule shall be emplaced into a stainless steel ampoule, which shall be welded. In exceptional cases the stainless steel ampoule may only be hermetically screwed. If no organic materials are utilised for sealing of the stainless steel ampoule, its dimensions and thickness shall be sufficient to endure, without damage, the gas pressure generated as a result of the radioactive decay of radium. The sealed stainless steel ampoule shall be emplaced into a lead container which shall be emplaced into the middle of a 0.2 m³ steel drum and the empty space between the container and the drum (also beneath and above the lead container) shall be filled with concrete.

64. Spent sealed neutron radiation sources shall be emplaced into a stainless steel ampoule, which shall be welded. In exceptional cases the ampoule may only be hermetically screwed. If no organic materials are utilised for sealing of the stainless steel ampoule, its dimensions and thickness shall be sufficient to endure, without damage, the gas pressure generated as a result of the radioactive decay. The sealed stainless steel ampoule shall be emplaced into a lead container which shall be emplaced into the middle of a 0.2 m³ steel drum and the empty space between the container and the drum (also beneath and above the lead container) shall be filled with concrete.

65. Spent sealed alpha radiation sources, which do not contain radium, in conformity with their total radioactivity shall be packaged:

65.1. into a stainless steel container which shall be filled with heated quartz sand and shall be emplaced into special sockets in a reinforced concrete container; or

65.2. in a stainless steel or plastic container. In exceptional cases (if the total radioactivity of the sources after the end of the State supervision period has decreased as a result of the radioactive decay of radionuclides to the limits for the non-exceeding of which no special permit (licence) or permit is required) the spent sealed radiation source may be packaged into at least three special plastic bags which shall be emplaced into the middle of a 0.2 m³ steel drum and the empty space between the bag or container and the drum (also beneath and above the bag or container) shall be filled with concrete.

66. Radioactive waste, which contains ion exchange resins, shall be processed so that the absorbed and adsorbed radioactive substances are stabilised in the processed radioactive waste by means of the cementation method. If the operator carries out the processing of radioactive waste, the relevant cementation technology shall be co-ordinated with the operator of the radioactive waste disposal or management undertaking.

VII. Requirements for Near-Surface Repository and Geological Repository of Radioactive Waste

67. When selecting a location for a near-surface repository or geological repository (hereinafter — disposal site) the following criteria shall be taken into account:

67.1. the climate and the annual amount of precipitation (average values, maximum and minimum observed values);

67.2. the economic activity to be performed in the surrounding territory;

67.3. the density of population;

67.4. the ability of the environment to absorb the impact caused by planned operations with radioactive waste without negative changes;

67.5. mineral resources;

67.6. hydrology of surface water and ground water;

67.7. seismic activities and karst formations;

67.8. potential possibilities for the industrial development; and

67.9. the proximity of other countries.

68. Within the controlled area of the radioactive waste disposal or management undertaking (within a territory around the disposal site where the received ionising radiation dose may reach 20 mSv per year) there shall be:

68.1. a part of the disposal site where it is possible to dispose packaged radioactive waste (hereinafter — radioactive waste vault);

68.2. long-lived radioactive waste storage vault or a room where radioactive waste may be permanently stored until the transfer thereof for disposal in the geological repository;

68.3. a temporary radioactive waste repository for short-term storage of spent sealed radiation sources;

68.4. a temporary radioactive waste repository where radioactive waste shall be stored until their treatment, processing and preparation for disposal;

- 68.5. decontamination site or room; and
- 68.6. a room for storage of working materials and equipment.

69. Within the supervision area of the radioactive waste disposal or management undertaking (within a territory outside the controlled area where there is a possibility that the received effective dose may exceed 1 mSv) there shall be:

- 69.1. a security centre;
- 69.2. a parking place for means of transport;
- 69.3. an administrative building, a maintenance building and a laboratory;
- 69.4. a warehouse for materials, objects and spare parts;
- 69.5. a water supply system (including a pump station and water treatment plants);
- 69.6. a site for washing of means of transport;
- 69.7. an access road connecting the supervision area with the controlled area;
- 69.8. a municipal sewage water treatment system; and
- 69.9. a boiler house and electric power supply system.

70. A near-surface repository shall consist of several radioactive waste vaults, which shall be made of reinforced concrete, and whose minimum volume for disposal of radioactive waste shall be 100 m³ and which may be divided into separate compartments.

71. When planning and creating radioactive waste vaults at a radioactive waste disposal or management undertaking the operator shall ensure that the contact of radioactive waste vaults with ground waters is prevented and a precipitation collection system is developed to drain them from the disposal site and ensure a possibility to control radioactive pollution in these waters.

72. The minimum wall thickness of radioactive waste vaults shall be 40 cm unless a thicker physical barrier layer is required in conformity with the long-term safety assessment. In assessing the necessary thickness of the physical barrier, both the natural physical barriers (including surrounding soil and ground layers) and the engineering structures which prevent or hinder the potential migration of radioactive waste from its disposal site shall be taken into account.

73. Radioactive waste to which storage until decay may be applied shall not be disposed of in a near-surface repository but long-lived radioactive waste may be stored therein until it is possible to dispose them in a geological repository.

74. In a near-surface repository the following shall be disposed of:

74.1. radioactive waste which does not contain isotopes with a half-life longer than 30 years;

74.2. radioactive waste whose total radioactivity is smaller than the values specified in Annex 3 to these Regulations;

74.3. low and intermediate radioactivity waste which contains only such radionuclides whose total or specific radioactivity after the time period during which the control in the radiation safety and nuclear safety area is performed and radiation monitoring programmes are implemented in order to ascertain that the radioactive waste does not endanger the environment, workers carrying out operations with radioactive waste and the population (hereinafter — State

supervision period), as a result of their decay will have decreased to the limits specified in regulatory enactments for the non-exceeding of which no special permit (licence) or permit is required; and

74.4. radioactive waste containing radionuclides whose total and specific radioactivity after the end of the State supervision period as a result of their decay exceeds the limits specified in regulatory enactments for the non-exceeding of which no special permit (licence) or permit is required if the long-term safety assessment proves that after an additional physical barrier has been created and the empty spaces in the radioactive waste vault between radioactive waste packages are filled with concrete, the potential ionising radiation dose caused by radioactive waste does not exceed 30 μSv per year.

75. In a geological repository the following shall be disposed of:

75.1. radioactive waste containing isotopes with a half-life longer than 30 years;

75.2. radioactive waste whose total radioactivity is greater than the values specified in Annex 3 of these Regulations;

75.3. low and intermediate radioactivity waste which contains radionuclides whose total or specific radioactivity after the end of the State supervision period is greater than the limits specified in regulatory enactments for the non-exceeding of which no special permit (licence) or permit is required; and

75.4. high radioactivity waste.

76. For a near-surface repository the State supervision period shall include the supervision of the near-surface repository until the moment when the acceptance of radioactive waste in the radioactive waste disposal or management undertaking is fully terminated (the undertaking is closed down) and 300 years after its closure.

77. The maximum permissible total radioactivity of radioactive waste in a package and in a radioactive waste vault, when disposing thereof in a near-surface repository, is prescribed in Annex 3 of these Regulations. In determining the maximum radioactivity for radioactive waste packages, safe transportation of radioactive waste shall be ensured and one of the following conditions shall be observed:

77.1. after the end of the State supervision period the radioactivity of radionuclides due to their decay will have decreased to the limits specified in regulatory enactments for the non-exceeding of which no special permit (licence) or permit is required; or

77.2. the limit specified in Annex 3 of these Regulations for a specific radionuclide may be exceeded if by means of the long-term safety assessment it is possible to substantiate that the foreseeable ionising radiation dose created by radioactive waste shall not exceed 300 μSv per year.

VIII. Requirements for Acceptance of Radioactive Waste for Disposal or Long-term Storage

78. The operator shall:

78.1. be responsible for radioactive waste until the moment when it is placed in the specialised means of transport of the radioactive waste disposal or management undertaking or are accepted for storage if a means of transport of another undertaking is utilised; and

78.2. provide the radioactive waste disposal or management undertaking with all the necessary information (including the radioactive waste container and radioactive waste treatment and processing quality control documents regarding radioactive waste) and provide a possibility to become acquainted with administrative, radiation safety, nuclear safety and technical capacity for the transfer of radioactive waste for disposal within the operator-controlled area on site.

79. The work supervisor of the radioactive waste disposal or management undertaking or another person working in the radiation safety and nuclear safety division of this undertaking upon acceptance of the radioactive waste from the operator shall check:

79.1. the ionising radiation dose rate at a distance of 1 cm and 1 m from the surface of the packaging; and

79.2. the radioactive contamination of the package on the surface thereof.

80. If it has been co-ordinated with the radioactive waste disposal or management undertaking, the maximum permissible ionising radiation dose rate on the external surface of the radioactive waste package may be 10 mSv/h but the maximum transport index may not exceed 10.

81. When accepting radioactive waste for disposal or long-term storage, the work supervisor of the radioactive waste disposal or management undertaking shall verify:

81.1. whether the dimensions and mass of the radioactive waste package, as well as auxiliary devices utilised for lifting and moving of such package conform to the requirements specified by the radioactive waste disposal or management undertaking;

81.2. the information provided by the operator regarding radioactive waste in each radioactive waste package with respect to the following:

81.2.1. the compliance of the technology utilised for treatment and processing with these Regulations; and

81.2.2. the compliance with disposal or long-term storage requirements; and

81.3. whether the deed of acceptance and delivery of radioactive waste specifies sufficient information regarding the radioactive waste (also regarding all radionuclides, their specific and total radioactivity), its treatment and processing, as well as regarding the packaging of the radioactive waste.

82. Safety of reinforced concrete containers shall be assessed by determining the tritium and ^{137}Cs diffusion speed through a sample of concrete utilised in manufacturing of the radioactive waste containers the diameter of which sample shall be 7 cm and the thickness — 1 cm (for tritium) and 0.5 cm (for caesium).

83. Radioactive waste, which contains chemically or biologically hazardous materials, and materials, which may cause dangerous infectious diseases or in any other way endanger the epidemiological safety in the radioactive waste disposal or management undertaking shall not be accepted for disposal.

84. Radioactive waste which contains radionuclides with a half-life less than 50 days shall not be accepted for disposal if as a result of their decay no long-lived daughter products are produced.

IX. Requirements for Long-term Safety Assessment of Disposed Radioactive Waste

85. Long-term safety assessment for radioactive waste disposal in a near-surface repository shall be performed by analysing a 1000-year period, but in a geological repository — at least a 10 000-year period.

86. If there are no special considerations in respect of additional factors, which may affect the migration of radionuclides, the following radioactive substance pathways shall be analysed in the long-term safety assessment:

86.1. discharges from radioactive waste vaults into the ground layer;

86.2. migration through the ground layer to:

86.2.1. open water basins; and

86.2.2. ground water layers utilised for drinking water intake; and

86.3. utilisation of water:

86.3.1. for watering;

86.3.2. for human consumption (drinking water); and

86.3.3. for watering of domestic animals.

87. For radiation emergency cases additional pathways shall be assessed (including the production of radioactive material dust in explosions resulting from a subversive activity or a plane crash) and the probability of such possible emergency and its impact on the safety of radioactive waste storage shall be analysed.

88. In performing the long-term safety assessment of geological repositories the following hypothetical earthquake parameters shall be utilised:

88.1. the maximum magnitude — 5.4 on the Richter scale;

88.2. earthquake probability — once in 400 years;

88.3. horizontal acceleration — 2.4 m/s^2 ;

88.4. vertical acceleration — 1.2 m/s^2 ; and

88.5. vertical movement — 2.0 cm.

89. If on the basis of the long-term safety assessment it is determined that the foreseeable ionising radiation dose for the population who live in the immediate proximity of the radioactive waste management or disposal undertaking is:

89.1. 100 mSv per year or greater, measures shall be obligatorily taken to reduce the ionising radiation dose to approximately 300 μSv per year;

89.2. from 10 to 100 mSv per year, measures shall be taken within the next five years to reduce the ionising radiation dose to approximately 300 μSv per year; and

89.3. 10 mSv per year or smaller, taking of intervention measures shall be evaluated in conformity with the financial and technical possibilities.

90. If on the basis of the environmental radiation monitoring results it is determined that the foreseeable ionising radiation dose for the population who live in the immediate proximity of the radioactive waste management or disposal undertaking is:

90.1. 5 mSv per year or greater, measures shall be obligatorily taken to reduce the ionising radiation dose to approximately 300 μSv per year;

90.2. from 1 to 5 mSv per year, measures shall be taken within the next five years to reduce the ionising radiation dose to approximately 300 μ Sv per year; and

90.3. 1 mSv per year or smaller, taking of the necessary measures shall be evaluated in conformity with the financial and technical possibilities.

91. In the long-term safety assessment at least three following events shall be utilised which pose a danger to people and the environment:

91.1. transfer of radionuclides by water;

91.2. direct exposure; and

91.3. destruction of the radioactive waste vault and transfer of radionuclides by air (when constructing a road over the disposal site).

92. In analysing the possible transfer of radionuclides by water, the following assumptions shall be utilised:

92.1. complete degradation of the hydro-isolating layer takes place on the fiftieth year after the end of the State supervision period (350th year after the closure of a near-surface repository), hence the filtration speed through the hydro-isolating layer equals to the average speed in the ground layers of the relevant territory;

92.2. for the time period until the 350th year it shall be assumed that the water filtration speed through the hydro-isolating layers is 1% of the average filtration speed in the relevant ground layers;

92.3. walls of steel drums shall not be considered a barrier layer or it shall be assumed that the corrosion speed is 3 μ m per year;

92.4. the foreseeable ionising radiation dose shall be calculated for a hypothetical group of the population living at a distance of 1000 m from the disposal site; and

92.5. migration of radionuclides from a radioactive waste vault to ground waters and further transfer of radionuclides to the human organism by water if water is utilised as drinking water, for watering of gardens and for watering of domestic animals. Thereafter the other radioactive substance pathway shall be analysed — by dust generated by the soil polluted as a result of the watering of gardens.

93. In assessing direct exposure, the following main assumptions shall be utilised:

93.1. at the end of the State supervision period (300 years after the closure of the near-surface repository) an inhabitant builds a single family house directly on the reinforced concrete cover above a radioactive waste vault (the upper layer of soil has been removed);

93.2. an inhabitant stays in the house 6575 hours per year, outside the house — 2192 hours per year;

93.3. water for drinking and individual farming is obtained from the closest (in respect of the radioactive waste vault) ground water layer; and

93.4. the floor of the building is located three meters above the radioactive waste layer.

94. In assessing road construction, the following main assumptions shall be utilised:

94.1. operations take place after the end of the State supervision period (300 years after the closure of a near-surface repository);

94.2. road construction works are carried out with the average speed of 10 km per six months;

- 94.3. work is carried out eight hours per day 20 days per month;
- 94.4. radioactive waste vaults are partly or fully destroyed because the intervention depth is six meters;
- 94.5. dust is directly inhaled; and
- 94.6. radioactive dust together with food from hands is transferred to the organism.

95. Operations and events which pose a danger to people and the environment during the operational and State supervision period and which are utilised in the safety assessment shall be the following:

- 95.1. a radiation emergency during performance of technological operations with radioactive waste packages;
- 95.2. an emergency during performance of technological operations in relation with labelling, concreting and dismantling of buildings;
- 95.3. a plane crash that partly or fully damages the cover of radioactive waste vaults;
- 95.4. an explosion (caused by subversive activity) which partly or fully damages the cover of radioactive waste vaults;
- 95.5. seismic activities (including karst formations) which may partly damage the cover of radioactive waste vaults;
- 95.6. roots of plants and trees and other biological effect which reduces hydro-isolation of the cover of radioactive waste vaults; and
- 95.7. precipitation.

96. Operations and events which pose a danger to people and the environment after the end of the State supervision period and which are utilised in the long-term safety assessment shall be the following:

- 96.1. road construction; and
- 96.2. building construction;
- 96.3. a plane crash that partly damages the cover of radioactive waste vaults;
- 96.4. seismic activity (including karst formations) which may partly damage the cover of radioactive waste vaults;
- 96.5. roots of plants and trees and other biological effect which reduces hydro-isolation of the cover of radioactive waste vaults; and
- 96.6. precipitation.

X. Requirements for Radioactive Waste Management After Closure of Radioactive Waste Vault And Termination of Disposal Site Operation

97. After the filling of near-surface radioactive waste vaults with radioactive waste packages preservation of radioactive waste shall be performed. Preservation of radioactive waste has the following stages:

97.1. the first stage shall commence before covering of vaults with a ground layer and shall end 30 years after creation of the vault protection layer and stabilisation of the ground layer. The principal measures:

97.1.1. physical protection of the territory shall be ensured, utilising also guard patrols directly around the radioactive waste vaults;

97.1.2. a supervision territory around the radioactive waste management or disposal undertaking shall be determined;

97.1.3. environmental radiation monitoring shall be ensured;

97.1.4. radionuclide radioactivity measurements of ground water and drainage water, as well as other examinations shall be performed to ascertain that there is no radionuclide leakage from radioactive waste vaults; and

97.1.5. ground water control wells and other systems shall be maintained in order;

97.2. the second stage shall commence 30 years after the creation of the radioactive waste vault protection layer and stabilisation of the ground layer and shall end 260 years thereafter. The principal measures — the measures referred to in Clauses 97.1.1, 97.1.2, 97.1.3 and 97.1.4 of these Regulations; and

97.3. the third stage shall commence 260 years after the creation of a radioactive waste vault protection layer and stabilisation of the ground layer and shall last until the end of the State supervision period. The principal measures — the measures referred to in Clauses 97.1.1 and 97.1.3 of these Regulations, as well as imposing of a prohibition on earth excavation works and other operations which may affect the safety of the radioactive waste vaults.

98. Prior to the closure of a radioactive waste vault the empty spaces in the radioactive waste vault shall be filled:

98.1. with concrete if at the end of the State supervision period in the near-surface repository there are radioactive waste packages in which the specific radioactivity of radionuclides exceeds the limits prescribed in regulatory enactments for the non-exceeding of which no special permit (licence) or permit is required, and in the whole radioactive waste vault the average specific radioactivity is smaller than the relevant limits; or

98.2. with sand if at the end of the State supervision period in the near-surface repository in all radioactive waste packages the specific radioactivity of radioactive waste as a result of radionuclide decay is less than the relevant limits and placing the relevant additional layers specified in Paragraph 99 of these Regulations above it, except for the case when the liquidation concept provides for dismantling of radioactive waste vaults and transfer of the radioactive waste packages and disposal thereof in municipal waste disposal sites.

99. In order to ensure that the amount of atmospheric precipitation, which reaches the radioactive waste vault, does not exceed 1.5 litres per 1 m² per year and that there is minimum human interference possible, after closure of the radioactive waste vault a multiple protection system shall be created:

99.1. above the radioactive waste vault at least 1 m thick monolith reinforced concrete layer shall be created with a slope at the sides of the radioactive waste vault to prevent accumulation of water above the radioactive waste vault;

99.2. above the reinforced concrete layer a multi-layer hydro-isolating cover shall be created;

99.3. to prevent damage of hydro-isolating cover at least 1 m thick gravel layer shall be spread on top of it;

99.4. above the gravel layer a 50 cm thick clay protection layer shall be created in addition;

99.5. above the clay layer at least 1 m thick crushed stone layer shall be created to protect the radioactive waste vault against human intervention; and

99.6. a 2 m thick soil layer shall be created last.

100. The layer covering the radioactive waste vault shall have the following technical requirements:

100.1. rain water filtration through the layer to the reinforced concrete layer covering the radioactive waste vault shall be reduced to the minimum;

100.2. the covering layer shall retain its properties unchangeable under all weather conditions possible in the relevant territory. In assessment the maximum parameters observed during the last 100 years shall be utilised; and

100.3. the covering layer shall endure freezing and thawing cycles, biological effect (also the effect of the tree root system growing above the radioactive waste vaults) after the end of the State supervision period until the moment when the total and specific radioactivity in all radioactive waste packages will have decreased to the limits prescribed by regulatory enactments for the non-exceeding of which no special permit (licence) or permit is required.

101. After closure of a radioactive waste vault a labelling in the reinforced concrete layer shall be made which shall warn of the disposed radioactive waste. In making the labelling of radioactive waste vaults the following requirements shall be met:

101.1. the labelling shall be made directly in the reinforced concrete layer, utilising stone squares in a contrast colour;

101.2. the labelling shall contain an inscription "RADIOAKTĪVIE ATKRITUMI" [radioactive waste] and the internationally accepted radiation symbol;

101.3. the inscription shall be clearly legible, the radiation symbol — unambiguous;

101.4. the inscription shall be made in four languages — Latvian, English, German and Russian; and

101.5. the year when the relevant vault was sealed shall be specified below the inscription (using both Arabic and Roman numerals).

102. When levelling off the disposal site, additional markers shall be placed beneath the ground which shall have the labelling hollowed out and each side of the triangular pyramid shall bear an inscription "RADIOAKTĪVIE ATKRITUMI" [radioactive waste] in Latvian and in one of the foreign languages specified in Sub-paragraph 101.4 of these Regulations and the base — the radiation symbol. The additional markers shall be placed as follows:

102.1. in the depth of 1 m from the ground surface a marker in the form of a triangular pyramid shall be placed, the linear dimensions of which shall be 0.5 m. The distance between such markers shall be approximately 5 m; and

102.2. in the depth of 0.3 m from the ground surface markers whose linear dimensions are 0.2 m shall be placed. The distance between such markers shall be approximately 2.5 m.

103. Before the end of the State supervision period a concrete pyramid shall be placed above the ground layer which covers the radioactive waste vault, in each corner and in the middle of the vault on which there shall be the labelling specifying the information referred to in Paragraph 101 of these Regulations and the number of the radioactive waste vault in conformity with the record information.

104. After the closure of a radioactive waste disposal or management undertaking the work performance schedule shall be the following:

104.1. during the time period until the 285th year after closure of the radioactive waste disposal or management undertaking, environmental radiation monitoring, repair of radioactive waste packages and packaging of radioactive waste packages in outer additional packages shall be performed if such is necessary, as well as physical protection shall be ensured, if necessary also utilising guards patrols;

104.2. on the 285th year after the closure of the radioactive waste disposal or management undertaking the total and specific radioactivity for all radioactive waste packages shall be calculated as it will be on the 300th year after the closure of the undertaking referred to and the state of the packages shall be examined. Damaged radioactive waste packages shall be packaged additionally (for example, 0.2 m³ steel drums shall be emplaced into 0.25 m³ steel drums and the whole empty space between the drums (also beneath and above the 0.2 m³ drum) shall be filled with concrete);

104.3. on the 286th year the empty spaces in radioactive waste vaults shall be filled with sand or concrete, taking into account the provisions specified in Paragraph 98 of these Regulations;

104.4. on the 290th year additional filling and compacting in the radioactive waste tanks shall be performed if necessary, the covering reinforced concrete layer and all additional layers shall be created;

104.5. on the 295th year buildings shall be dismantled, the upper ground layer shall be adjusted, a level surface shall be created above all radioactive waste vaults in the disposal site and concrete markers shall be placed;

104.6. on the 300th year the soil layer shall be adjusted, additional concrete markers shall be placed by perimeter and in the disposal site; and

104.7. on the 301st year State supervision shall be terminated.

XI. Requirements for Return of Spent Ionising Radiation Sources to Manufacturers and Shipment of Radioactive Waste for Processing Outside Latvia

105. The operator who plans to import a spent ionising radiation source which contains radioactive materials with such total radioactivity that after the use of these ionising radiation sources for 10 years it will exceed 100 MBq, before acquiring thereof in the operator's ownership or possession shall take all the possible measures so that the possibility to return such spent ionising radiation source to its manufacturer is provided for in the purchase and sales or gift contract.

106. The purchase and sales or gift contract shall at least provide for the following provisions:

106.1. the maximum time limit during which the spent ionising radiation source may be returned to its supplier shall be 15 years after entering into the purchase and sales or gift contract; and

106.2. the supplier shall undertake to accept back a spent ionising radiation source after the end of the safe usage period thereof within a year after receipt of a written request by the ionising radiation source user.

107. Extracts from the purchase and sales or gift contract regarding the possibility to return a spent ionising radiation source to its supplier shall be submitted to the Centre and the State institution responsible for radiation safety and nuclear safety where the supplier is registered.

108. It shall be prohibited to ship radioactive waste to:

108.1. places which are located further than 60° in the southern hemisphere;

108.2. those African, Caribbean and Pacific States which on 15 December 1989 in Lome signed the Fourth African, Caribbean and Pacific - European Economic Community Convention; and

108.3. states that do not have sufficient technical, legal or administrative capacity to safely carry out operations with radioactive waste without endangering the environment and the population.

109. If the requirements set out in Paragraph 108 of these Regulations are not complied with, the Centre shall ensure that the radioactive waste is returned to that operator whose operations have produced it.

110. If shipment of radioactive waste for processing outside Latvia is planned, the operator shall submit to the Centre an application for receipt of the relevant authorisation (Section one of Annex 4). In the application the operator may also request authorisation for several shipments of radioactive waste if:

110.1. radioactive waste in all planned shipments will have the same physical and chemical properties, it will contain the same radionuclides and its total and specific radioactivity will be the same;

110.2. the radioactive waste shipments in all cases are planned by one and the same operator, one and the same carrier will be utilised and there will be one and the same radioactive waste processor;

110.3. in all radioactive waste transportation cases transit countries will be the same and the same border crossing points will be utilised; and

110.4. the total shipment period for all radioactive waste shall not be longer than three years.

111. The Centre shall send the application and an authorisation form of the competent institution (sample of the authorisation — in Section two of Annex 4) to the competent institution of that state in which the processing of radioactive waste is planned to be performed and to the competent institutions of all those state through which it is planned to transport such radioactive waste, as well as request provision of an opinion in respect of the application.

112. If official authorisation is received from all competent institutions of those states to which the application to authorise shipment of radioactive waste for processing outside Latvia was sent, the Centre may issue an authorisation for the radioactive waste export for processing (sample of the authorisation — Section three of Annex 4).

113. A copy of the authorisation issued by the Centre shall be sent to the radioactive waste processor together with other necessary documents specified in Cabinet Regulation No 307 of 3

July 2001, Regulations Regarding Protection Against Ionising Radiation During Transportation of Radioactive Materials and other regulatory enactments.

114. The undertaking to which radioactive waste has been shipped for processing shall within a period of 15 days after receipt of the radioactive waste at the processing site send to the competent institution of its state an acknowledgement (Sections four and five of Annex 4) regarding receipt of such radioactive waste but to all competent institutions of those states which were involved in the authorisation of such consignment — copies of the acknowledgement.

115. The procedures referred to in Paragraphs 108, 109, 110, 111, 112, 113 and 114 of these Regulations shall not be applied if spent sealed radiation sources (except for nuclear materials) in conformity with the purchase and sales or gift contracts are returned to the manufacturers thereof.

116. The operator may not refuse to accept the radioactive waste back if due to any reason the shipment of radioactive waste cannot be completed.

XII. Closing Provision

117. The Environmental Protection and Regional Development Ministry in accordance with the procedures set out in law shall authorise an institution, which until creation of a geological repository shall store all long-lived radioactive waste.

Prime Minister

A. Bērziņš

Minister for Environmental Protection and Regional Development

V. Makarovs

Minimum Significant Specific Radioactivity

Table 1

Minimum Significant Specific Radioactivity for Various Materials

No	Radionuclide	Radioactive contamination on the surface if materials are used again (Bq/cm ²)	Specific radioactivity if concrete construction debris is used (Bq/g)	Specific radioactivity on a municipal or hazardous waste landfill site or dump if materials related to radioactive waste are disposed of (Bq/g)
1.	³ H	1 x 10 ⁴	3 x 10 ⁴	1 x 10 ⁵
2.	¹⁴ C	1 x 10 ²	3 x 10 ²	1 x 10 ³
3.	²² Na	1 x 10 ⁻¹	3 x 10 ⁻¹	1 x 10 ⁰
4.	²⁴ Na	1 x 10 ⁻¹	3 x 10 ⁻¹	1 x 10 ⁰
5.	³² P	1 x 10 ⁻¹	3 x 10 ¹	1 x 10 ²
6.	³⁵ S	1 x 10 ³	3 x 10 ³	1 x 10 ⁴
7.	³⁶ Cl	1 x 10 ²	3 x 10 ²	1 x 10 ³
8.	⁴⁵ Ca	1 x 10 ²	3 x 10 ²	1 x 10 ³
9.	⁵¹ Cr	1 x 10 ¹	3 x 10 ¹	1 x 10 ²
10.	⁵⁴ Mn	1 x 10 ⁰	3 x 10 ⁻¹	1 x 10 ⁰
11.	⁵⁵ Fe	1 x 10 ²	3 x 10 ²	1 x 10 ³
12.	⁵⁹ Fe	1 x 10 ⁰	3 x 10 ⁻¹	1 x 10 ⁰
13.	⁵⁷ Co	1 x 10 ¹	3 x 10 ⁰	1 x 10 ¹
14.	⁵⁸ Co	1 x 10 ⁰	3 x 10 ⁻¹	1 x 10 ⁰
15.	⁶⁰ Co	1 x 10 ⁰	3 x 10 ⁻¹	1 x 10 ⁰
16.	⁶³ Ni	1 x 10 ³	3 x 10 ³	1 x 10 ⁴
17.	⁶⁵ Zn	1 x 10 ¹	3 x 10 ⁻¹	1 x 10 ⁰
18.	⁸⁹ Sr	1 x 10 ¹	3 x 10 ¹	1 x 10 ²
19.	⁹⁰ Sr	1 x 10 ⁰	3 x 10 ⁰	1 x 10 ¹
20.	⁹⁰ Y	1 x 10 ⁰	3 x 10 ¹	1 x 10 ²
21.	⁹⁴ Nb	1 x 10 ⁰	3 x 10 ⁻¹	1 x 10 ⁰
22.	^{99m} Tc	1 x 10 ⁰	3 x 10 ⁰	1 x 10 ¹
23.	⁹⁹ Tc	1 x 10 ¹	3 x 10 ²	1 x 10 ³
24.	¹⁰⁶ Ru	1 x 10 ¹	3 x 10 ⁰	1 x 10 ¹

25.	110mAg	1 x 10 ⁰	3 x 10 ⁻¹	1 x 10 ⁰
26.	109Cd	1 x 10 ²	3 x 10 ²	1 x 10 ³
27.	111In	1 x 10 ⁰	3 x 10 ⁰	1 x 10 ¹
28.	124Sb	1 x 10 ⁰	3 x 10 ⁻¹	1 x 10 ⁰
29.	125I	1 x 10 ¹	3 x 10 ¹	1 x 10 ²
30.	129I	1 x 10 ⁰	3 x 10 ⁰	1 x 10 ¹
31.	131I	1 x 10 ¹	3 x 10 ⁰	1 x 10 ¹
32.	134Cs	1 x 10 ⁰	3 x 10 ⁻¹	1 x 10 ⁰
33.	137Cs	1 x 10 ⁰	3 x 10 ⁻¹	1 x 10 ⁰
34.	144Ce	1 x 10 ⁰	3 x 10 ⁰	1 x 10 ¹
35.	147Pm	1 x 10 ²	3 x 10 ²	1 x 10 ³
36.	152Eu	1 x 10 ⁰	3 x 10 ⁻¹	1 x 10 ⁰
37.	192Ir	1 x 10 ⁰	3 x 10 ⁻¹	1 x 10 ⁰
38.	204Tl	1 x 10 ¹	3 x 10 ⁰	1 x 10 ¹
39.	210Pb	1 x 10 ⁻¹	3 x 10 ⁻¹	1 x 10 ⁰
40.	210Po	1 x 10 ⁻¹	3 x 10 ⁻¹	1 x 10 ⁰
41.	226Ra	1 x 10 ⁻¹	3 x 10 ⁻¹	1 x 10 ⁰
42.	228Ra	1 x 10 ⁻¹	3 x 10 ⁻¹	1 x 10 ⁰
43.	228Th	1 x 10 ⁻¹	3 x 10 ⁻²	1 x 10 ⁻¹
44.	230Th	1 x 10 ⁻¹	3 x 10 ⁻²	1 x 10 ⁻¹
45.	232Th	1 x 10 ⁻¹	3 x 10 ⁻²	1 x 10 ⁻¹
46.	234U	1 x 10 ⁻¹	3 x 10 ⁻¹	1 x 10 ⁰
47.	235U	1 x 10 ⁻¹	3 x 10 ⁻¹	1 x 10 ⁰
48.	238U	1 x 10 ⁻¹	3 x 10 ⁻¹	1 x 10 ⁰
49.	237Np	1 x 10 ⁻¹	3 x 10 ⁻²	1 x 10 ⁻¹
50.	239Pu	1 x 10 ⁻¹	3 x 10 ⁻²	1 x 10 ⁻¹
51.	240Pu	1 x 10 ⁻¹	3 x 10 ⁻²	1 x 10 ⁻¹
52.	241Pu	1 x 10 ⁻¹	3 x 10 ⁰	1 x 10 ¹
53.	241Am	1 x 10 ⁻¹	3 x 10 ⁻²	1 x 10 ⁻¹
54.	244Cm	1 x 10 ⁻¹	3 x 10 ⁻¹	1 x 10 ⁰

Table 2

Minimum Significant Specific Radioactivity for Scrap Metal

No.	Radionuclide	Specific radioactivity for scrap metal containing iron (Bq/g)	Specific radioactivity for scrap metal containing copper (Bq/g)	Specific radioactivity for scrap metal containing aluminium (Bq/g)
1.	^3H	1×10^3	9×10^4	2×10^4
2.	^{14}C	8×10^1	3×10^3	6×10^2
3.	^{22}Na	2×10^{-1}	2×10^0	4×10^{-1}
4.	^{24}Na	6×10^{-1}	6×10^{-1}	6×10^{-1}
5.	^{32}P	2×10^2	2×10^2	2×10^2
6.	^{35}S	6×10^2	1×10^4	3×10^3
7.	^{36}Cl	1×10^1	3×10^2	4×10^2
8.	^{45}Ca	6×10^2	6×10^2	1×10^3
9.	^{51}Cr	7×10^1	7×10^1	7×10^1
10.	^{54}Mn	2×10^0	4×10^0	4×10^0
11.	^{55}Fe	3×10^4	3×10^4	7×10^4
12.	^{59}Fe	2×10^0	2×10^0	2×10^0
13.	^{57}Co	2×10^1	2×10^2	3×10^1
14.	^{58}Co	1×10^0	4×10^0	4×10^0
15.	^{60}Co	6×10^{-1}	1×10^0	1×10^0
16.	^{63}Ni	3×10^5	2×10^4	1×10^5
17.	^{65}Zn	5×10^{-1}	5×10^0	5×10^0
18.	^{89}Sr	6×10^1	6×10^1	6×10^1
19.	^{90}Sr	1×10^1	9×10^0	4×10^1
20.	^{90}Y	9×10^1	3×10^1	9×10^1
21.	^{94}Nb	4×10^{-1}	9×10^{-1}	6×10^{-1}
22.	$^{99\text{m}}\text{Tc}$	2×10^1	2×10^1	2×10^1
23.	^{99}Tc	4×10^1	4×10^2	5×10^2
24.	^{106}Ru	1×10^0	7×10^0	9×10^0
25.	$^{110\text{m}}\text{Ag}$	5×10^{-1}	5×10^{-1}	1×10^0
26.	^{109}Cd	1×10^1	3×10^2	2×10^2
27.	^{111}In	6×10^0	6×10^0	6×10^0
28.	^{124}Sb	8×10^{-1}	8×10^{-1}	5×10^{-1}
29.	^{125}I	3×10^0	1×10^2	1×10^2
30.	^{129}I	4×10^{-1}	2×10^1	6×10^1

31.	^{131}I	6×10^0	6×10^0	6×10^0
32.	^{134}Cs	2×10^{-1}	2×10^0	6×10^{-1}
33.	^{137}Cs	6×10^{-1}	7×10^0	2×10^0
34.	^{144}Ce	1×10^1	2×10^1	2×10^1
35.	^{147}Pm	6×10^3	7×10^4	3×10^4
36.	^{152}Eu	5×10^{-1}	3×10^0	8×10^{-1}
37.	^{192}Ir	2×10^0	6×10^0	4×10^0
38.	^{204}Tl	3×10^2	4×10^2	5×10^2
39.	^{210}Pb	7×10^{-2}	1×10^0	3×10^{-1}
40.	^{210}Po	2×10^0	2×10^1	2×10^0
41.	^{226}Ra	4×10^{-1}	9×10^{-1}	5×10^{-1}
42.	^{228}Ra	7×10^{-1}	2×10^0	1×10^0
43.	^{228}Th	4×10^{-1}	1×10^0	6×10^{-1}
44.	^{230}Th	3×10^{-1}	2×10^0	4×10^0
45.	^{232}Th	3×10^{-1}	2×10^0	4×10^0
46.	^{234}U	3×10^0	7×10^0	2×10^0
47.	^{235}U	3×10^0	8×10^0	8×10^{-1}
48.	^{238}U	4×10^0	8×10^0	2×10^0
49.	^{237}Np	6×10^{-1}	3×10^0	7×10^0
50.	^{239}Pu	2×10^{-1}	2×10^0	3×10^0
51.	^{240}Pu	3×10^{-1}	2×10^0	3×10^0
52.	^{241}Pu	1×10^1	8×10^1	2×10^2
53.	^{241}Am	3×10^{-1}	2×10^0	4×10^0
54.	^{244}Cm	6×10^{-1}	3×10^0	6×10^0

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Maximum Permissible Total and Specific Radioactivity if Radioactive Waste is Transported in 0.1 m³ Steel Drums Without Additional Treatment (LSA-III)

No	Radionuclide	Maximum permissible specific radioactivity (Bq/kg)	Maximum permissible total radioactivity for a package (Bq)
1.	³ H	4 x 10 ¹¹	1 x 10 ¹³
2.	⁷ Be	2 x 10 ¹¹	1 x 10 ¹²
3.	¹⁴ C	3 x 10 ⁷	1 x 10 ⁹
4.	²² Na	5 x 10 ⁹	1 x 10 ¹⁰
5.	³⁵ S	3 x 10 ¹⁰	1 x 10 ¹²
6.	³⁶ Cl	1 x 10 ⁵	1 x 10 ⁶
7.	⁴⁰ K	1 x 10 ⁶	1 x 10 ⁷
8.	⁴⁵ Ca	1 x 10 ¹⁰	1 x 10 ¹²
9.	⁴⁶ Sc	1 x 10 ⁹	1 x 10 ¹⁰
10.	⁵¹ Cr	3 x 10 ¹¹	1 x 10 ¹²
11.	⁵³ Mn	1 x 10 ⁷	1 x 10 ⁹
12.	⁵⁴ Mn	1 x 10 ¹⁰	1 x 10 ¹²
13.	⁵⁵ Fe	1 x 10 ¹¹	1 x 10 ¹²
14.	⁵⁶ Co	1 x 10 ⁹	1 x 10 ¹⁰
15.	⁵⁷ Co	1 x 10 ¹¹	1 x 10 ¹²
16.	⁵⁸ Co	1 x 10 ¹⁰	1 x 10 ¹¹
17.	⁶⁰ Co	1 x 10 ⁹	1 x 10 ¹⁰
18.	⁵⁹ Ni	1 x 10 ⁷	1 x 10 ⁸
19.	⁶³ Ni	1 x 10 ⁸	1 x 10 ⁹
20.	⁶⁵ Zn	6 x 10 ⁹	1 x 10 ¹¹
21.	⁷³ As	4 x 10 ¹¹	1 x 10 ¹²
22.	⁷⁵ Se	1 x 10 ¹⁰	1 x 10 ¹¹
23.	⁸¹ Kr	1 x 10 ⁸	1 x 10 ⁹
24.	⁸⁵ Kr	1 x 10 ¹¹	1 x 10 ¹²
25.	⁸⁵ Sr	1 x 10 ¹⁰	1 x 10 ¹¹
26.	⁸⁹ Sr	6 x 10 ⁹	1 x 10 ¹¹
27.	⁹⁰ Sr ⁺	3 x 10 ⁸	1 x 10 ⁹
28.	⁹¹ Y	6 x 10 ⁹	1 x 10 ¹¹

29.	$^{93}\text{Zr}^+$	1×10^7	1×10^8
30.	^{95}Zr	8×10^9	1×10^{11}
31.	$^{93\text{m}}\text{Nb}$	3×10^{11}	1×10^{12}
32.	^{94}Nb	1×10^5	1×10^6
33.	^{93}Mo	1×10^7	1×10^8
34.	^{97}Tc	1×10^7	1×10^8
35.	$^{97\text{m}}\text{Tc}$	1×10^{11}	1×10^{12}
36.	^{99}Tc	1×10^7	1×10^8
37.	$^{106}\text{Ru}^+$	2×10^9	1×10^{10}
38.	$^{110\text{m}}\text{Ag}$	1×10^9	1×10^{10}
39.	^{109}Cd	2×10^{10}	1×10^{12}
40.	^{113}Sn	1×10^{10}	1×10^{11}
41.	^{124}Sb	6×10^9	1×10^{11}
42.	^{125}Sb	1×10^{10}	1×10^{11}
43.	$^{123\text{m}}\text{Te}$	1×10^{10}	1×10^{12}
44.	$^{125\text{m}}\text{Te}$	9×10^9	1×10^{12}
45.	$^{127\text{m}}\text{Te}$	7×10^9	1×10^{12}
46.	^{125}I	3×10^{10}	1×10^{12}
47.	^{129}I	1×10^6	1×10^7
48.	^{134}Cs	7×10^9	1×10^{11}
49.	^{135}Cs	1×10^8	1×10^9
50.	$^{137}\text{Cs}^+$	1×10^9	1×10^{10}
51.	^{139}Ce	2×10^{10}	1×10^{12}
52.	$^{144}\text{Ce}^+$	1×10^9	1×10^{10}
53.	^{151}Sm	1×10^8	1×10^9
54.	^{152}Eu	1×10^9	1×10^{10}
55.	^{154}Eu	6×10^9	1×10^{11}
56.	^{155}Eu	3×10^{10}	1×10^{12}
57.	^{153}Gd	9×10^{10}	1×10^{12}
58.	^{160}Tb	6×10^9	1×10^{11}
59.	^{170}Tm	6×10^9	1×10^{11}
60.	^{171}Tm	1×10^{10}	1×10^{11}
61.	^{182}Ta	5×10^9	1×10^{11}
62.	^{181}W	3×10^{11}	1×10^{12}
63.	^{185}W	8×10^9	1×10^{12}
64.	^{185}Os	1×10^{10}	1×10^{11}
65.	^{192}Ir	5×10^9	1×10^{12}
66.	^{204}Tl	7×10^9	1×10^{12}

67.	$^{210}\text{Pb}^+$	1×10^8	1×10^9
68.	^{207}Bi	1×10^7	1×10^8
69.	^{210}Po	2×10^9	1×10^{12}
70.	$^{222}\text{Rn}^+$	4×10^7	1×10^{10}
71.	$^{226}\text{Ra}^+$	1×10^5	1×10^6
72.	$^{228}\text{Ra}^+$	1×10^9	1×10^{10}
73.	$^{228}\text{Th}^+$	1×10^7	1×10^{10}
74.	$^{229}\text{Th}^+$	1×10^4	1×10^5
75.	^{230}Th	1×10^4	1×10^5
76.	dab.Th	1×10^4	1×10^5
77.	^{231}Pa	1×10^4	1×10^5
78.	$^{232}\text{U}^+$	1×10^5	1×10^6
79.	^{233}U	1×10^4	1×10^5
80.	^{234}U	1×10^4	1×10^5
81.	$^{235}\text{U}^+$	1×10^4	1×10^5
82.	^{236}U	1×10^4	1×10^5
83.	$^{238}\text{U}^+$	1×10^4	1×10^5
84.	$^{237}\text{Np}^+$	1×10^4	1×10^5
85.	^{236}Pu	1×10^3	1×10^4
86.	^{238}Pu	1×10^4	1×10^5
87.	^{239}Pu	1×10^4	1×10^5
88.	^{240}Pu	1×10^4	1×10^5
89.	^{241}Pu	1×10^5	1×10^6
90.	^{242}Pu	1×10^5	1×10^6
91.	^{244}Pu	1×10^4	1×10^5
92.	^{241}Am	1×10^4	1×10^5
93.	$^{242\text{m}}\text{Am}^+$	1×10^4	1×10^5
94.	$^{243}\text{Am}^+$	1×10^4	1×10^5
95.	^{242}Cm	1×10^6	1×10^7
96.	^{243}Cm	1×10^6	1×10^7
97.	^{244}Cm	1×10^6	1×10^7
98.	^{245}Cm	1×10^4	1×10^5
99.	^{246}Cm	1×10^4	1×10^5
100.	^{247}Cm	1×10^4	1×10^5
101.	^{248}Cm	1×10^4	1×10^5
102.	^{249}Bk	1×10^5	1×10^6
103.	^{248}Cf	1×10^6	1×10^7
104.	^{249}Cf	1×10^4	1×10^5

105.	^{250}Cf	1×10^5	1×10^6
106.	^{251}Cf	1×10^4	1×10^5
107.	^{252}Cf	1×10^7	1×10^8
108.	^{253}Cf	1×10^6	1×10^7
109.	^{254}Cf	1×10^7	1×10^8

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Maximum Permissible Total Radioactivity of Radioactive Waste Upon Disposal Thereof in Near-Surface Repository (in a Vault, for Package and Spent Sealed Radiation Source)

No	Radionuclide	Maximum permissible total radioactivity in a 100 m ³ radioactive waste vault (Bq)	Maximum permissible total radioactivity for a 0.2 m ³ radioactive waste package (Bq)	Maximum permissible total radioactivity for a spent sealed radiation source (Bq)
1.	³ H	1 x 10 ¹⁷	1 x 10 ¹³	1 x 10 ⁷
2.	⁷ Be	no limit	1 x 10 ¹³	3 x 10 ¹⁰
3.	¹⁴ C	1 x 10 ¹³	1 x 10 ¹⁰	1 x 10 ⁸
4.	²² Na	no limit	1 x 10 ¹¹	3 x 10 ¹⁰
5.	³⁵ S	no limit	1 x 10 ¹³	3 x 10 ¹¹
6.	³⁶ Cl	3 x 10 ¹²	5 x 10 ⁹	1 x 10 ⁷
7.	⁴⁰ K	1 x 10 ¹¹	1 x 10 ⁹	1 x 10 ⁸
8.	⁴⁵ Ca	no limit	1 x 10 ¹³	3 x 10 ¹¹
9.	⁴⁶ Sc	no limit	1 x 10 ¹¹	3 x 10 ¹⁰
10.	⁵¹ Cr	no limit	1 x 10 ¹³	3 x 10 ¹⁰
11.	⁵³ Mn	1 x 10 ¹²	1 x 10 ¹⁰	1 x 10 ⁶
12.	⁵⁴ Mn	no limit	1 x 10 ¹²	3 x 10 ⁸
13.	⁵⁵ Fe	no limit	4 x 10 ¹³	1 x 10 ¹²
14.	⁵⁶ Co	no limit	3 x 10 ¹¹	2 x 10 ¹⁰
15.	⁵⁷ Co	no limit	1 x 10 ¹³	2 x 10 ¹²
16.	⁵⁸ Co	no limit	1 x 10 ¹²	2 x 10 ¹¹
17.	⁶⁰ Co	1 x 10 ²²	1 x 10 ¹¹	1 x 10 ¹¹
18.	⁵⁹ Ni	1 x 10 ¹³	1 x 10 ¹⁰	3 x 10 ⁸
19.	⁶³ Ni	1 x 10 ¹⁴	1 x 10 ¹¹	3 x 10 ⁸
20.	⁶⁵ Zn	no limit	1 x 10 ¹²	1 x 10 ¹²
21.	⁷³ As	no limit	4 x 10 ¹³	3 x 10 ¹⁰
22.	⁷⁵ Se	no limit	3 x 10 ¹²	3 x 10 ¹⁰
23.	⁸¹ Kr	1 x 10 ¹³	1 x 10 ¹⁰	3 x 10 ⁷
24.	⁸⁵ Kr	1 x 10 ¹⁹	1 x 10 ¹³	1 x 10 ⁸
25.	⁸⁵ Sr	no limit	2 x 10 ¹²	3 x 10 ¹⁰
26.	⁸⁹ Sr	no limit	1 x 10 ¹²	3 x 10 ¹⁰
27.	⁹⁰ Sr+	1 x 10 ¹¹	1 x 10 ¹⁰	3 x 10 ⁹

28.	^{91}Y	no limit	1×10^{12}	3×10^{10}
29.	$^{93}\text{Zr}^+$	1×10^{12}	1×10^{10}	3×10^8
30.	^{95}Zr	no limit	1×10^{12}	3×10^{10}
31.	$^{93\text{m}}\text{Nb}$	1×10^{15}	1×10^{13}	3×10^6
32.	^{94}Nb	1×10^9	1×10^7	3×10^6
33.	^{93}Mo	1×10^{12}	1×10^{10}	3×10^8
34.	^{97}Tc	1×10^{12}	1×10^{10}	3×10^6
35.	$^{97\text{m}}\text{Tc}$	1×10^{14}	1×10^{13}	3×10^6
36.	^{99}Tc	3×10^{12}	5×10^9	1×10^7
37.	$^{106}\text{Ru}^+$	no limit	2×10^{11}	3×10^{10}
38.	$^{110\text{m}}\text{Ag}$	no limit	1×10^{11}	1×10^{11}
39.	^{109}Cd	no limit	3×10^{13}	3×10^{11}
40.	^{113}Sn	no limit	1×10^{13}	3×10^{10}
41.	^{124}Sb	no limit	1×10^{12}	1×10^{11}
42.	^{125}Sb	no limit	1×10^{12}	1×10^{11}
43.	$^{123\text{m}}\text{Te}$	1×10^{20}	1×10^{13}	3×10^{10}
44.	$^{125\text{m}}\text{Te}$	no limit	2×10^{13}	3×10^{10}
45.	$^{127\text{m}}\text{Te}$	no limit	2×10^{13}	3×10^{10}
46.	^{125}I	no limit	1×10^{13}	1×10^{10}
47.	^{129}I	5×10^{10}	5×10^7	3×10^6
48.	^{134}Cs	no limit	1×10^{12}	1×10^{11}
49.	^{135}Cs	1×10^{13}	1×10^{10}	7×10^4
50.	$^{137}\text{Cs}^+$	1×10^{12}	3×10^{10}	3×10^9
51.	^{139}Ce	no limit	1×10^{13}	3×10^{10}
52.	$^{144}\text{Ce}^+$	no limit	1×10^{11}	7×10^{10}
53.	^{151}Sm	1×10^{13}	1×10^{10}	3×10^9
54.	^{152}Eu	1×10^{13}	1×10^{12}	3×10^{10}
55.	^{154}Eu	1×10^{16}	1×10^{12}	7×10^{10}
56.	^{155}Eu	no limit	1×10^{13}	7×10^{10}
57.	^{153}Gd	no limit	1×10^{13}	7×10^{10}
58.	^{160}Tb	no limit	1×10^{12}	3×10^{10}
59.	^{170}Tm	no limit	1×10^{12}	3×10^{10}
60.	^{171}Tm	no limit	1×10^{13}	7×10^{10}
61.	^{182}Ta	no limit	1×10^{12}	3×10^{10}
62.	^{181}W	no limit	3×10^{13}	3×10^{11}
63.	^{185}W	no limit	4×10^{13}	3×10^{11}
64.	^{185}Os	no limit	1×10^{12}	1×10^{11}
65.	^{192}Ir	no limit	1×10^{13}	3×10^{12}

66.	^{204}Tl	no limit	1×10^{13}	3×10^{10}
67.	$^{210}\text{Pb}^+$	1×10^{12}	1×10^{11}	3×10^8
68.	^{207}Bi	1×10^{12}	1×10^{10}	3×10^8
69.	^{210}Po	no limit	1×10^{13}	3×10^9
70.	$^{222}\text{Rn}^+$	1×10^{12}	1×10^{11}	3×10^9
71.	$^{226}\text{Ra}^+$	1×10^9	5×10^6	4×10^5
72.	$^{228}\text{Ra}^+$	1×10^{19}	1×10^{11}	3×10^9
73.	$^{228}\text{Th}^+$	no limit	5×10^{11}	3×10^8
74.	$^{229}\text{Th}^+$	3×10^8	1×10^6	2×10^5
75.	^{230}Th	3×10^8	1×10^6	2×10^5
76.	$^{232}\text{Th}_{\text{sec}}$	3×10^8	1×10^6	2×10^5
77.	^{231}Pa	3×10^8	1×10^6	2×10^5
78.	$^{232}\text{U}^+$	3×10^8	1×10^6	1×10^6
79.	^{233}U	3×10^8	1×10^6	1×10^6
80.	^{234}U	3×10^8	1×10^6	1×10^6
81.	$^{235}\text{U}^+$	3×10^8	1×10^6	3×10^5
82.	^{236}U	3×10^8	1×10^6	1×10^6
83.	$^{238}\text{U}^+$	3×10^8	1×10^6	1×10^6
84.	$^{237}\text{Np}^+$	3×10^8	1×10^6	4×10^4
85.	^{236}Pu	1×10^8	1×10^6	3×10^5
86.	^{238}Pu	3×10^8	1×10^6	3×10^5
87.	^{239}Pu	3×10^8	1×10^6	3×10^5
88.	^{240}Pu	3×10^8	1×10^6	3×10^5
89.	^{241}Pu	1×10^8	1×10^7	3×10^5
90.	^{242}Pu	3×10^8	1×10^6	3×10^5
91.	^{244}Pu	3×10^8	1×10^6	3×10^5
92.	^{241}Am	3×10^8	1×10^6	3×10^5
93.	$^{242\text{m}}\text{Am}^+$	3×10^8	1×10^6	4×10^4
94.	$^{243}\text{Am}^+$	3×10^8	1×10^6	4×10^4
95.	^{242}Cm	1×10^9	1×10^8	1×10^6
96.	^{243}Cm	1×10^9	1×10^8	1×10^6
97.	^{244}Cm	1×10^9	1×10^8	1×10^6
98.	^{245}Cm	3×10^8	1×10^6	4×10^4
99.	^{246}Cm	3×10^8	1×10^6	4×10^4
100.	^{247}Cm	3×10^8	1×10^6	4×10^4
101.	^{248}Cm	3×10^8	1×10^6	4×10^4

10 2.	249Bk	1×10^8	1×10^7	1×10^6
10 3.	248Cf	1×10^{10}	1×10^8	3×10^5
10 4.	249Cf	3×10^8	1×10^6	4×10^4
10 5.	250Cf	1×10^8	1×10^7	3×10^5
10 6.	251Cf	3×10^8	1×10^6	4×10^4
10 7.	252Cf	1×10^{10}	1×10^9	3×10^5
10 8.	254Cf	1×10^{12}	1×10^9	1×10^6

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All documents the samples of which are given in this Annex shall be printed on white A4 format paper in black letters. In the international transport the English language shall be used and the usage of another official language of the United Nations Organisation understood by all persons involved in the transportation shall be permissible.

STANDARD DOCUMENT FOR CONTROL OF SHIPMENTS OF RADIOACTIVE WASTE

Section 1

Registration number _____
(to be completed by the Radiation Safety Centre)

APPLICATION FOR RADIOACTIVE WASTE SHIPMENT AUTHORISATION

1.	<p>Type of shipment ¹</p> <p>Shipment to an EU Member State (Type B) <input type="checkbox"/></p> <p>Receipt from an EU Member State (Type C) <input type="checkbox"/></p> <p>Transit through an EU Member State (Type D) <input type="checkbox"/></p> <p>Shipment to a state which is not an EU Member State (Type E) <input type="checkbox"/></p> <p>Receipt from a state which is not an EU Member State (Type F) <input type="checkbox"/></p>
2.	<p>Application for authorisation for</p> <p>A single shipment <input type="checkbox"/></p> <p>Several shipments <input type="checkbox"/> Number of shipments planned _____</p> <p>Planned period of execution _____</p>
3.	<p>Radioactive waste producer _____ (name)</p> <p>_____</p> <p>Contact person _____</p> <p>Address _____</p> <p>Post code _____ Town _____ Country _____</p> <p>Telephone _____ Fax _____</p>

4.	(To be completed if the radioactive waste is not with the producer thereof) Place where radioactive waste is located Address _____ Post code _____ Town _____ Country _____ Telephone _____ Fax _____
5.	Description of radioactive waste _____ _____ Physical and chemical characteristics _____ Main radionuclides _____ Maximum alpha radioactivity in a package (GBq) _____ Maximum beta radioactivity in a package (GBq) _____ Maximum gamma radioactivity in a package (GBq) _____
6.	Total alpha radioactivity (GBq) Total beta radioactivity (GBq) Total gamma radioactivity (GBq) Total number of packages *Total weight of radioactive waste (kg) *Total weight with packaging (kg) *Total volume (additional information) (*To be completed if the application for authorisation is intended for several shipments) Description of the radioactive waste package (including plastic bags, 200l steel drums, ISO transport containers) _____ Information on labelling of packages (attach the samples of the used labelling, stickers, posters) _____ _____
7.	Other hazardous properties Category 1 Explosive substances and products <input type="checkbox"/> Category 2 Compressed, liquefied or dissolved under pressure gases <input type="checkbox"/> Category 3 Inflammable liquids <input type="checkbox"/> Category 4 4.1. Inflammable solids and materials <input type="checkbox"/> 4.2. Substances liable to spontaneous combustion <input type="checkbox"/> 4.3. Substances which, on contact with water, give off inflammable gases <input type="checkbox"/> Category 5 5.1. Oxidising substances <input type="checkbox"/> 5.2. Organic peroxides <input type="checkbox"/>

Category 6	6.1. Poisonous substances	<input type="checkbox"/>
	6.2. Infectious substances	<input type="checkbox"/>
Category 8	Corrosive substances	<input type="checkbox"/>
Category 9	Other dangerous substances and products	<input type="checkbox"/>


8.	Description of causes giving rise to radioactive waste (medical or industrial usage, scientific research or other cause) <hr/> <hr/>			
9.	Purpose of the shipment Radioactive waste from processing of spent nuclear fuel <input type="checkbox"/> Shipment of radioactive waste for processing or packaging <input type="checkbox"/> Receipt of radioactive waste after processing or packaging <input type="checkbox"/> Shipment of radioactive waste for temporary storage <input type="checkbox"/> Receipt of radioactive waste after temporary storage <input type="checkbox"/> Shipment of radioactive waste for disposal <input type="checkbox"/> Receipt of equivalent amount of radioactive waste in connection with disposal thereof outside the Republic of Latvia <input type="checkbox"/> Other purposes <input type="checkbox"/> <hr/> <input type="checkbox"/> <hr/> <input type="checkbox"/>			
10.	Planned form of transport (road, rail, air, sea vessels, river vessels)	Point of departure	Point of destination	Planned carrier
	1. _____ 2. _____ 3. _____ 4. _____ 5. _____	_____ _____ _____ _____ _____	_____ _____ _____ _____ _____	_____ _____ _____ _____ _____
11.	Countries in the order of shipment performance (the first shall be indicated the country where the radioactive waste is held and the last the country to which it is shipped)			
	1. _____ 2. _____	3. _____ 4. _____	5. _____ 6. _____	7. _____ 8. _____
12.	Consignee of radioactive waste at the point of destination _____ (legal name) <hr/> Contact person _____ Address _____ Post code _____ Town _____ Country _____ Telephone _____ Fax _____			

STANDARD DOCUMENT FOR CONTROL OF RADIOACTIVE WASTE SHIPMENTS

Section 2

Registration number _____

AUTHORISATION FOR SHIPMENT BY THE COMPETENT INSTITUTION

16.	<p>Type of shipment</p> <p>Type B Shipment to an EU Member State <input type="checkbox"/></p> <p>Type C Receipt from an EU Member State <input type="checkbox"/></p> <p>Type D Transit through an EU Member State <input type="checkbox"/></p> <p>Type E Shipment to a state which is not an EU Member State <input type="checkbox"/></p> <p>Type F Receipt from a state which is not an EU Member State <input type="checkbox"/></p> <p>Authorisation issuer³ _____ (legal name)</p> <p>Contact person _____</p> <p>Address _____</p> <p>Post code _____ Town _____ Country _____</p> <p>Telephone _____ Fax _____</p>
17.	<p>Date of registration of the submission _____</p> <p>_____</p> <p>(signature and full name)</p> <p>Place for a seal</p> 
18.	<p>Competent institution⁴</p> <p>Country _____</p> <p>Place of origin of the waste <input type="checkbox"/> Transit country <input type="checkbox"/> Country of destination <input type="checkbox"/></p> <p>Name of the competent institution _____</p> <p>Contact person _____</p>

	Address _____		
	Post code _____	Town _____	Country _____
	Telephone _____	Fax _____	

19.	Approval of the radioactive waste shipment by the competent institution⁵			
	Authorised	<input type="checkbox"/>	Conditions if any	<input type="checkbox"/>
	Not authorised	<input type="checkbox"/>	Reasons for the refusal	<input type="checkbox"/>
	Conditions		Reasons for the refusal	
	_____		_____	
	_____		_____	
	_____		_____	
	_____		_____	
	_____		_____	
	_____		_____	
	_____		_____	
	_____		_____	
	_____		_____	
	_____		_____	
_____		_____		
(date and place)		<div style="border: 1px dashed black; border-radius: 50%; width: 150px; height: 100px; margin: 0 auto; display: flex; align-items: center; justify-content: center;"> <div style="text-align: center;">Place for a seal</div> </div>		
_____		_____		
(signature and full name)				

Notes.

³ The issuer of the authorisation subsequent to receipt of an application for radioactive waste shipment authorisation (hereinafter — authorisation) shall without delay complete boxes 16 and 17 of this documents and register the application for the authorisation, as well as prepare the necessary number of authorisation copies for sending to all competent institutions of those states from which consent is required (hereinafter — competent institutions). For each competent institution box 18 shall be completed. The issuer of the authorisation shall prepare a copy of Section 1 and 2 of the document and send it to the relevant competent institution to receive its opinion.

⁴ The competent institution may supplement or adjust the information specified in box 18 of this document.

⁵ Within two months after receipt of these documents the competent institution shall complete box 19 of this document and shall send the original of the copy of Section 2 of

this document back to the issuer of the authorisation. The time for taking of the decision may be extended by one month if the competent institution requests such.

STANDARD DOCUMENT FOR CONTROL OF RADIOACTIVE WASTE SHIPMENTS

Section 3

Registration number _____
(to be completed by the Radiation Safety Centre)

RADIOACTIVE WASTE SHIPMENT AUTHORISATION

20.	<p>Issuer of the authorisation⁶ in conformity with the type of shipment</p> <p>Type B Radiation Safety Centre <input type="checkbox"/></p> <p>Type C Radiation Safety Centre <input type="checkbox"/></p> <p>Type D Competent institution of an EU Member State <input type="checkbox"/></p> <p>Type E Competent institution of the country (which is not an EU Member State) <input type="checkbox"/></p> <p>Type F Radiation Safety Centre <input type="checkbox"/></p> <p>Legal name of the competent institution _____</p> <p>_____</p> <p>Contact person _____</p> <p>Address _____</p> <p>Post code _____ Town _____ Country _____</p> <p>Telephone _____ Fax _____</p>																																						
21.	<p>Authorisation granted <input type="checkbox"/> For a single shipment <input type="checkbox"/></p> <p>Authorisation not granted <input type="checkbox"/> For several shipments <input type="checkbox"/></p> <p>Authorisation valid until _____</p>																																						
22.	<p>Countries in the order of shipment performance (the first shall be specified the country where the waste is located but the last the country to which it is shipped)</p> <table border="1" style="width: 100%; border-collapse: collapse;"> <thead> <tr> <th rowspan="2" style="width: 25%;">Countries</th> <th colspan="2" style="width: 15%;">Conditions</th> <th rowspan="2" style="width: 25%;">Countries</th> <th colspan="2" style="width: 15%;">Conditions</th> </tr> <tr> <th style="width: 5%;">yes</th> <th style="width: 5%;">no</th> <th style="width: 5%;">yes</th> <th style="width: 5%;">no</th> </tr> </thead> <tbody> <tr> <td>1. _____</td> <td style="text-align: center;"><input type="checkbox"/></td> <td style="text-align: center;"><input type="checkbox"/></td> <td>5. _____</td> <td style="text-align: center;"><input type="checkbox"/></td> <td style="text-align: center;"><input type="checkbox"/></td> </tr> <tr> <td>2. _____</td> <td style="text-align: center;"><input type="checkbox"/></td> <td style="text-align: center;"><input type="checkbox"/></td> <td>6. _____</td> <td style="text-align: center;"><input type="checkbox"/></td> <td style="text-align: center;"><input type="checkbox"/></td> </tr> <tr> <td>3. _____</td> <td style="text-align: center;"><input type="checkbox"/></td> <td style="text-align: center;"><input type="checkbox"/></td> <td>7. _____</td> <td style="text-align: center;"><input type="checkbox"/></td> <td style="text-align: center;"><input type="checkbox"/></td> </tr> <tr> <td>4. _____</td> <td style="text-align: center;"><input type="checkbox"/></td> <td style="text-align: center;"><input type="checkbox"/></td> <td>8. _____</td> <td style="text-align: center;"><input type="checkbox"/></td> <td style="text-align: center;"><input type="checkbox"/></td> </tr> </tbody> </table>					Countries	Conditions		Countries	Conditions		yes	no	yes	no	1. _____	<input type="checkbox"/>	<input type="checkbox"/>	5. _____	<input type="checkbox"/>	<input type="checkbox"/>	2. _____	<input type="checkbox"/>	<input type="checkbox"/>	6. _____	<input type="checkbox"/>	<input type="checkbox"/>	3. _____	<input type="checkbox"/>	<input type="checkbox"/>	7. _____	<input type="checkbox"/>	<input type="checkbox"/>	4. _____	<input type="checkbox"/>	<input type="checkbox"/>	8. _____	<input type="checkbox"/>	<input type="checkbox"/>
Countries	Conditions		Countries	Conditions																																			
	yes	no		yes	no																																		
1. _____	<input type="checkbox"/>	<input type="checkbox"/>	5. _____	<input type="checkbox"/>	<input type="checkbox"/>																																		
2. _____	<input type="checkbox"/>	<input type="checkbox"/>	6. _____	<input type="checkbox"/>	<input type="checkbox"/>																																		
3. _____	<input type="checkbox"/>	<input type="checkbox"/>	7. _____	<input type="checkbox"/>	<input type="checkbox"/>																																		
4. _____	<input type="checkbox"/>	<input type="checkbox"/>	8. _____	<input type="checkbox"/>	<input type="checkbox"/>																																		

23.	<p style="text-align: center;">Conditions (indicating the country imposing the conditions and indicate references to attached documents)</p> <p>_____</p> <p>_____</p> <p>_____</p> <p>_____</p>	<p style="text-align: center;">Reason for the refusal</p> <p>_____</p> <p>_____</p> <p>_____</p> <p>_____</p>
24.	<p>Competent institutions are informed that the radioactive waste shipment authorisation has been granted or refused.</p> <div style="text-align: center; border: 1px dashed black; border-radius: 50%; width: 150px; height: 100px; margin: 20px auto;"> <p>Place for a seal</p> </div> <p style="text-align: center;">_____</p> <p style="text-align: center;">(date and place) _____ (signature and full name)</p>	
<p>P.S.</p> <p>1. Authorisation shall not release the radioactive waste keeper, carrier, producer, consignee or any other natural person or legal person involved in the shipment of the radioactive waste from liability in respect of failure to perform the relevant requirements.</p> <p>2. When shipping radioactive waste, appropriately completed and approved Sections 1, 3 and 4 of the standard document shall be used as freight accompanying documents.</p>		

Notes.

⁶ The issuer of the authorisation:

a) when completing box 21 the condition shall be taken into account that the maximum period of validity of the authorisation is three years;

b) shall send the authorisation to the applicant thereof (the person who has applied for radioactive waste shipment or receipt authorisation), attaching to the document Sections 1, 4 and 5 of this document;

c) shall send copies of this document to the competent institutions.


STANDARD DOCUMENT FOR CONTROL OF RADIOACTIVE WASTE SHIPMENTS

Section 4

Registration number _____
(to be completed by the Radiation Safety Centre)

LIST OF RADIOACTIVE WASTE PACKAGES⁷

25.	<p>Applicant of the authorisation _____ (legal name)</p> <p>_____</p> <p>Contact person _____</p> <p>Address _____</p> <p>Post code _____ Town _____ Country _____</p> <p>Telephone _____ Fax _____</p>
26.	<p>Authorisation granted for a single <input type="checkbox"/></p> <p>shipment</p> <p>Authorisation granted for several <input type="checkbox"/></p> <p>shipments</p> <p style="text-align: right;">Serial number of the shipment _____</p>
27.	<p>Description of radioactive waste _____</p> <p>_____</p> <p>Physical and chemical characteristics _____</p> <p>Main radionuclides _____</p> <p>Maximum alpha radioactivity in a package (GBq) _____</p> <p>Maximum beta radioactivity in a package (GBq) _____</p> <p>Maximum gamma radioactivity in a package (GBq) _____</p> <p>Description of the waste packaging (for example, plastic bags, 200l steel drums, ISO transport containers) _____</p>
28.	<p>Total alpha radioactivity (GBq) _____</p> <p>Total beta radioactivity (GBq) _____</p> <p>Total gamma radioactivity (GBq) _____</p> <p>Total weight with the packaging (kg) _____</p> <p>Total volume (additional information) _____</p>

29.	<p>Information regarding labelling of radioactive waste packages (identification number of each package, total weight of the package (kg/package), weight of radioactive waste in the package (kg/package), radioactivity (GBq/package))</p> <p>_____</p> <p>_____</p> <p>_____</p> <p>_____</p> <p>_____</p> <p>_____</p> <p>_____</p> <p>_____</p> <p>_____</p> <p>_____</p> <p>_____</p>
<p>(If the space provided above is insufficient additional sheets shall be used or documents containing the necessary information shall be attached)</p>	
30.	<p>Date of dispatch _____</p> <p>I hereby certify that all the information provided in this document and in the attached documents (if any) is correct and accurate (to the best of my knowledge).</p> <div style="text-align: center; margin: 20px 0;">  <p>Place for a seal</p> </div> <p>_____</p> <p style="text-align: center;">(date and place)</p> <p style="text-align: right;">_____</p> <p style="text-align: right;">(signature and full name)</p>

Note.

⁷ The list shall be completed by the radioactive waste producer before each shipment (also if the authorisation has been granted for several shipments). This Section of the document (like Sections 1 and 3 of this document) shall be used as a freight accompanying document during the whole period of shipment. After completion of the shipment this document shall be attached to the acknowledgement of radioactive waste receipt.



**STANDARD DOCUMENT
FOR CONTROL OF RADIOACTIVE WASTE SHIPMENTS**

Section 5

Registration number _____
(to be completed by the Radiation Safety Centre)

ACKNOWLEDGEMENT OF RADIOACTIVE WASTE RECEIPT⁸

31.	<p>Consignee of radioactive waste at the point of destination _____ (legal name)</p> <hr/> <p>Contact person _____ Address _____ Post code _____ Town _____ Country _____ Telephone _____ Fax _____ Place where the radioactive waste is stored _____</p> <hr/> <p>Contact person _____ Address _____ Post code _____ Town _____ Country _____ Telephone _____ Fax _____</p>
32.	<p>Authorisation granted for a single shipment <input type="checkbox"/></p> <p>Authorisation granted for several shipments <input type="checkbox"/></p> <p style="text-align: right;">Serial number of the shipment _____</p> <p>It is the last shipment to which the authorisation is granted Yes <input type="checkbox"/> No <input type="checkbox"/></p>
33	<p>(To be completed only if the type of shipment is C, D or F)</p> <p style="text-align: center;">Border crossing point upon entry to the country to which the radioactive waste is shipped</p> <p>Country _____ Border crossing point _____</p>
34.	<p>Date of receipt of the radioactive waste _____</p> <p>Date when the acknowledgement is sent (together with Section 4 of the standard document) _____</p> <p>The acknowledgement shall be sent</p> <ul style="list-style-type: none"> - if the type of shipment is B or E, - to the competent institution of the state of destination of the radioactive waste; - if the type of shipment is C, D or E, - to the applicant of the authorisation.

	<p>I hereby certify that all the information provided in this document and in the attached documents (if any) is correct and accurate (to the best of my knowledge).</p> <div style="text-align: center; margin: 20px 0;">  <p>Place for a seal</p> </div> <div style="text-align: right; margin-top: 20px;"> <p>_____</p> <p>(signature and full name)</p> </div>
35.	<p>(To be completed only if the type of shipment is C, D or F)</p> <p>The applicant of the authorisation (sending the acknowledgement and radioactive waste consignee's declaration to the issuer of the authorisation):</p> <p>Date when the acknowledgement is sent (together with Section 4 of the standard document)</p> <p>_____</p> <p>Border crossing point upon entry to the country to which the radioactive waste is sent</p> <p>Country _____</p> <p>Border crossing point _____</p> <div style="text-align: center; margin: 20px 0;">  <p>Zīmoga vieta</p> </div> <div style="text-align: right; margin-top: 20px;"> <p>_____</p> <p>(signature and full name)</p> </div>
<p>P.S.</p> <ol style="list-style-type: none"> 1. If the consignee of the radioactive waste does not utilise this document, his or her declaration shall contain at least the same information as requested in boxes 31 and 35 of this document. 2. The competent institution, which has received the acknowledgement, shall send the copies thereof to other involved competent institutions. 3. Thereafter Sections 4 and 5 of this document shall be sent to the competent institution, which issued the radioactive waste shipment authorisation. 	

Note.

⁸ This Section shall be completed by the consignee of the radioactive waste, supplementing the information provided by the applicant of the authorisation. The type of acknowledgement shall be chosen in conformity with the type of shipment and the fact whether the authorisation has been granted for a single or several shipments.

If the authorisation is granted for a single shipment

1. Shipments of Type B or E

Within 15 days of receiving of the radioactive waste the consignee of the waste shall complete boxes 31, 32 and 34 of this document and send Sections 4 and 5 of this document to the competent institution of that country to which the radioactive waste has been shipped.

The competent institution of the country where the radioactive waste has been received shall send Sections 4 and 5 of this document to the competent institution, which issued the radioactive waste shipment authorisation, but a copy of Sections 4 and 5 of this document — to other involved competent institutions.

2. Shipment of type C, D or F

The consignee of the radioactive waste shall ensure that the consignor after receipt of the radioactive waste freight sends Sections 4 and 5 of this document (with completed boxes 31 and 34) without delay.

Within 15 days of receiving the radioactive waste the applicant of the authorisation shall send Sections 4 and 5 of this document to the competent institution which issued the radioactive waste shipment authorisation.

The competent institution, which issued the radioactive waste shipment authorisation, shall send Sections 4 and 5 of this document to other involved competent institutions.

If the authorisation is granted for several shipments

1. Shipments of type B or E

The consignee of radioactive waste shall after receipt of each shipment complete boxes 31, 32 and 34 of this document and send Sections 4 and 5 of this document to the competent institution of that country to which the radioactive waste is shipped. Prior to that the consignee of radioactive waste shall prepare the necessary number of uncompleted copies of Section 5.

2. Shipments of type C, D or F

The consignee of radioactive waste shall ensure that the consignor after receipt of each radioactive waste freight sends Sections 4 and 5 of this document (with completed boxes 31 and 34) without delay.

The applicant of the authorisation shall complete box 35 of this document and together with Section 4 of this document send it to the competent institution, which issued the radioactive waste shipment authorisation.

3. Radioactive waste shipments of any type

When all authorised radioactive waste shipments have been carried out the last acknowledgement of radioactive waste receipt shall be sent as if the authorisation were granted to a single shipment only, however:

— in box 32 of this document it shall be stated that it is the acknowledgement of the last authorised shipment;

— the notification shall state that all radioactive waste shipments have indeed been received; and

— in order to provide an overview, Section 4 of this document in respect of all shipments, which have been completed, shall be attached.

